

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

1 M D C C N 2 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 32

1 REPORT SOURCE L 6 0 5 0 0 0 3 1 8 7 1 2 2 5 7 8 8 0 1 2 4 7 9 9
 8 9 DOCKET NUMBER 62 63 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

1 During conduct of surveillance testing RCS unidentified leakage was determined
 2 to be 1.45 gpm at 0330. At 1040 the reactor was shutdown to facilitate
 3 containment entry, whereupon a cracked weld on 21A RCP upper seal pressure
 4 sensing line was discovered. The leak was immediately isolated. LERs 78-39
 5 (U-2) and 78-22 (U-1) describe similar events.
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1 SYSTEM CODE C B 11
 2 CAUSE CODE E 12
 3 CAUSE SUBCODE C 13
 4 COMPONENT CODE P I P E X X 14
 5 COMP. SUBCODE A 15
 6 VALVE SUBCODE Z 16
 7 LER/RO REPORT NUMBER 7 8 21 22
 8 SEQUENTIAL REPORT NO. 0 5 0 24 25
 9 OCCURRENCE CODE 0 3 26 27
 10 REPORT TYPE L 30
 11 REVISION NO. 0 32
 12 ACTION TAKEN Z 18 19
 13 FUTURE ACTION B 19
 14 EFFECT ON PLANT A 20
 15 SHUTDOWN METHOD A 21
 16 HOURS 0 0 1 8 37 40
 17 ATTACHMENT SUBMITTED Y 23
 18 NPD-4 FORM SUB. N 24
 19 PRIME COMP. SUPPLIER A 25
 20 COMPONENT MANUFACTURER B 5 8 0 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 Upon investigation the cracked weld was found to be located on the flange
 2 socket weld downstream of the second isolation valve. Weld repairs were
 3 performed for return to service. An engineering study is underway to propose
 4 design changes to correct the fatigue problem. (See attached sheet)
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1 FACILITY STATUS E 28
 2 % POWER 0 9 8 29
 3 OTHER STATUS NA 30
 4 METHOD OF DISCOVERY B 31
 5 DISCOVERY DESCRIPTION Surveillance Testing 32
 6 ACTIVITY CONTENT Z 33
 7 RELEASED OF RELEASE Z 34
 8 AMOUNT OF ACTIVITY NA 35
 9 LOCATION OF RELEASE NA 36
 10 PERSONNEL EXPOSURES NUMBER 0 0 1 0 37
 11 TYPE Z 38
 12 DESCRIPTION NA 39
 13 PERSONNEL INJURIES NUMBER 0 0 1 0 40
 14 DESCRIPTION NA 41
 15 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42
 16 DESCRIPTION NA 43
 17 PUBLICITY DESCRIPTION NA 45
 18 ISSUED Z 44
 19 NAME OF PREPARER J. Carroll/K. Watkins
 20 PHONE 301-234-7943/7960

7901300100

NRC USE ONLY

LER 78-50/3L
EVENT DATE 12/25/78
REPORT DATE 1/24/79
DOCKET NO. 50-318
ATTACHMENT

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS: (CONTINUED)

Upon investigation the cracked weld was found to be located on the flange socket weld downstream of the second isolation valve. Weld repairs were performed to return to service. A vibration analysis performed during the past refueling outage determined that fatigue levels of vibration existed at several locations on RCP instrument piping/tubing. Braces added to this piping have reduced but not eliminated the vibration. An engineering study is underway to propose design changes to correct the fatigue problem.