

LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 N Y J A F 1 2 0 0 - 0 0 0 0 - 0 0 0 0 3 4 1 1 1 1 1 1 4 5
7 8 9 14 15 25 26 30 57 CAT 58

01 REPORT SOURCE L 6 0 5 0 0 0 3 3 3 7 1 2 2 6 7 8 8 0 1 2 5 7 9 9
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
Please See Attachment

02
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09 SYSTEM CODE S A 11 CAUSE CODE A 12 CAUSE SUBCODE X 13 COMPONENT CODE Z Z Z Z Z Z 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 18 19 20
17 LER/RO REPORT NUMBER 7 8 21 22 23 24 26 27 28 29 30 31 32
EVENT YEAR 7 8 SEQUENTIAL REPORT NO. 1 1 2 OCCURRENCE CODE 0 3 REPORT TYPE L REVISION NO. 0
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS 22 ATTACHMENT SUBMITTED Y 23 NRPD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER Z 25 COMPONENT MANUFACTURER 7 9 9 9 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27
Please See Attachment

10
11
12
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14

15 FACILITY STATUS E 25 % POWER 0 2 8 29 OTHER STATUS NA 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Administrative Review 32
7 8 9 10 12 13 44 45 46 80
16 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36
7 8 9 10 11 44 45 80
17 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39
7 8 9 11 12 13 80
18 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41
7 8 9 11 12 80
19 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43
7 8 9 10 80
20 PUBLICITY ISSUED N 44 DESCRIPTION NA 45
7 8 9 10 80

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During Inspection 78-22, conducted October 4-6, 1978, the NRC Inspector noted that containment isolation valves 16-1-AOV-101 A and B and 16-1-AOV-102 A and B were not listed in the Technical Specifications for the FitzPatrick plant. These normal open valves were installed during the 1977 refueling outage in place of normally closed manual valves to allow continuous monitoring of drywell to suppression chamber differential pressure. The differential pressure is maintained at the FitzPatrick plant as required by the Nuclear Regulatory Commission as one of a number of actions taken to reduce the probability of damage to the Mark I containment structure in the event of certain accidents.

While a Plant Engineer was preparing a draft of a technical specification amendment to add the referenced valves to the technical specifications, it was noted that the surveillance requirements of Technical Specification Appendix A, Paragraph 4.7.D.1.a, 4.7.D.1.b, and 4.7.D.2 had not been performed on the referenced air operated isolation valves. However, following installation of the valves in 1977, pre-operational tests, to verify operability of remote manual and automatic closure, was completed. In addition, on several occasions since the pre-operational test, the valves have closed, as required, in response to automatic closure signals that were generated as a result of reactor scram and isolation.

Surveillance Tests associated with meeting the requirements of the referenced technical specification paragraphs are listed below:

- F-ST 1C Primary Containment Isolation Valve Exercise
- F-ST 1H Primary Containment Isolation Valve Inoperable Test
- F-ST 31B Drywell Isolation Valves, Atmosphere Control Valves, Sump Drain Valves and TIP Withdrawal Logic System Functional Test
- F-ST 34B Reactor Building Isolation Logic System Functional Test

Each of these procedures has been revised and operability and logic testing has been completed with satisfactory results.