

# OPERATING DATA REPORT

DOCKET NO. 50-321  
 DATE 04-10-84  
 COMPLETED BY: M. G. McBay  
 TELEPHONE (912) 367-7851

## OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 1
2. Reporting Period: 03-84
3. Licensed Thermal Power (Mwt): 2436
4. Nameplate Rating (Gross MWe): 809.3
5. Design Electrical Rating (Net MWe): 777.3
6. Maximum Dependable Capacity (Gross MWe): 801.2
7. Maximum Dependable Capacity (Net MWe): 752.2
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe)
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	2184	72311
12. Number of Hours Reactor was Critical	592.0	1515.6	51040.7
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	512.3	1419.5	47812.5
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1014102	3161286	100379314
17. Gross Electrical Energy Generated (MWH)	316770	1024280	32482450
18. Net Electrical Energy Generated (MWH)	299371	972904	30832267
19. Unit Service Factor	68.9	65.0	66.1
20. Unit Availability Factor	68.9	65.0	66.1
21. Unit Capacity Factor (Using MDC Net)	53.5	59.2	56.7
22. Unit Capacity Factor (Using DER Net)	51.8	57.3	54.9
23. Unit Forced Outage Rate	31.1	32.4	18.9
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shutdown at End of Report Period, Estimated Date of Startup:

26. Units in Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

8404160138 840331  
 PDR ADOCK 05000321  
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(9/77)

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-321  
 DATE: 04-10-84  
 COMPLETED BY: M. G. McBay  
 TELEPHONE (912) 367-7851

MONTH 03-84

DAY AVERAGE DAILY POWER LEVEL  
 (MWe-Net)

1	-6
2	-6
3	-6
4	-7
5	-7
6	-8
7	-8
8	279
9	310
10	186
11	-15
12	-14
13	101
14	315
15	405
16	562

DAY AVERAGE DAILY POWER LEVEL  
 (MWe-Net)

17	567
18	690
19	727
20	729
21	731
22	728
23	717
24	592
25	712
26	733
27	731
28	729
29	729
30	715
31	564

(9/77)

# OPERATING DATA REPORT

DUCKET NO. 50-366  
 DATE 04-10-84  
 COMPLETED BY: M. G. McBay  
 TELEPHONE (912) 367-7851

## OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 2
2. Reporting Period: 03-84
3. Licensed Thermal Power (Mwt): 2436
4. Nameplate Rating (Gross MWe): 817.0
5. Design Electrical Rating (Net MWe): 784.0
6. Maximum Dependable Capacity (Gross MWe): 803.9
7. Maximum Dependable Capacity (Net MWe): 747.9
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe):
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	2184	39937
12. Number of Hours Reactor was Critical	0.0	308.2	27379.4
13. Reactor Reserve Shutdown Hours	0.0	0	0
14. Hours Generator On-Line	0.0	308.2	26096.1
15. Unit Reserve Shutdown Hours	0.0	0	0
16. Gross Thermal Energy Generated (MWH)	0	726912	55943167
17. Gross Electrical Energy Generated (MWH)	0	242640	18414420
18. Net Electrical Energy Generated (MWH)	-2141	225917	17515024
19. Unit Service Factor	0.0	14.1	65.3
20. Unit Availability Factor	0.0	14.1	65.3
21. Unit Capacity Factor (Using MUC Net)	-0.4	13.8	58.6
22. Unit Capacity Factor (Using DER Net)	-0.4	13.2	55.9
23. Unit Forced Outage Rate	0.0	0.0	13.1
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shutdown at End of Report Period, Estimated Date of Startup:

26. Units in Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

# AVERAGE DAILY UNIT POWER LEVEL

DUCKET NO. 50-366  
 DATE: 04-10-84  
 COMPLETED BY: M. G. McBay  
 TELEPHONE (912) 367-7851

MONTH 03-84

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1	-3
2	-3
3	-3
4	-3
5	-3
6	-3
7	-3
8	-3
9	-3
10	-3
11	-3
12	-3
13	-3
14	-3
15	-3
16	-3

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17	-3
18	-3
19	-3
20	-3
21	-3
22	-3
23	-3
24	-3
25	-3
26	-3
27	-3
28	-2
29	-2
30	-2
31	-2

(9/77)



## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH

DOCKET NO. 50-321  
 UNIT NAME Hatch 1  
 DATE 04-10-84  
 COMPLETED BY M. G. McBay  
 TELEPHONE 912-367-7851

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
84-9	03-01-84	F	166.73	A	3	NA	HA	TURBIN	Reactor Scram from turbine high vibration. Inspection revealed 13th stage LP turbine buckets damaged.
84-10	03-08-84	S	4.57	H	3	NA	SA	VESSEL	Ramping up from Reactor Scram.
84-11	03-09-84	F	63.03	H	5	NA	CH	HTEXCH	Holding and Reducing load due to Feedwater Heater Problems.
84-12	03-10-84	F	65.39	A	1	NA	HA	TURBIN	Turbine manually tripped. Still trying to resolve feedwater heater problems.
84-13	03-11-84	F	13.60	H	2	NA	RC	CONROD	Reactor manually scrammed because of control rod pull error.
84-14	03-13-84	S	14.42	H	2	NA	SA	VESSEL	Ramping up from manual turbine trip.
84-15	03-15-84	S	25.00	H	5	NA	RC	INSTRU	Reducing and holding load for OD-1 test.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Continuations  
 5-Load Reduction  
 9-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup>  
 Exhibit I - Same Source

(0/77)

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH

DOCKET NO. 50-321  
 UNIT NAME Hatch 1  
 DATE 04-10-84  
 COMPLETED BY M. G. McBay  
 TELEPHONE 912-367-7851

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
84-16	03-15-84	S	42.54	H	5	NA	RC	INSTRU	Ramping to rated power from OD-1 testing.
84-17	03-16-84	S	3.13	B	5	NA	RC	CONROD	Reducing load for Rod Pattern Adjustment.
84-18	03-17-84	S	19.00	H	5	NA	RC	CONROD	Ramping back to rated power from Rod pattern adjustment.
84-19	03-23-84	S	3.66	H	5	NA	RC	CONROD	Reducing load for Rod Pattern Adjustment & weekly Turbine Test.
84-20	03-24-84	S	14.83	H	5	NA	RC	CONROD	Ramping back to rated power from Rod Adjustment and Turbine Test.
84-21	03-30-84	S	3.75	H	5	NA	RC	CONROD	Reducing load for Rod Pattern Adjustment & weekly Turbine Test.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Continuations  
 5-Load Reduction  
 9-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup>  
 Exhibit I - Same Source

(0/77)

# UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH

DOCKET NO. 50-321  
 UNIT NAME Hatch 1  
 DATE 04-10-84  
 COMPLETED BY M. G. McBay  
 TELEPHONE 912-367-7851

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
84-22	03-31-84	S	23.00	H	5	NA	RC	CONROD	Ramping back to rated power.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Continuations  
 5-Load Reduction  
 9-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MARCH

DOCKET NO. 50-366  
 UNIT NAME Hatch 2  
 DATE 04-10-84  
 COMPLETED BY M. G. McBay  
 TELEPHONE 912-367-7851

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
84-5	03-01-84	S	744	H	2	NA	CB	PIPEXX	Recirc Pipe Replacement Outage

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Continuations  
 5-Load Reduction  
 9-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup>  
 Exhibit I - Same Source

(9/77)



NARRATIVE REPORT  
UNIT 1

March 1st	0000	Unit outage continues. Outage is due to reactor scram from high turbine vibration on 2-11-84. Upon inspection found 13th stage LP turbine buckets severely damaged.
March 6th	0910	Reactor Mode switch to Start-up & Hot Standby.
March 6th	1828	Reactor critical.
March 7th	2216	Generator tied to line. Ramping up load.
March 8th	0030	Increasing load to 30%.
March 8th	2230	Unit holding at 345 MWE for Feedwater Heater Level problems.
March 9th	2246	Unit still holding. Trying to resolve heater drain pump problems.
March 10th	1501	Decreasing load to 100 MWE before tripping turbine to resolve heater drain valve problems.
March 10th	1802	Turbine manually tripped.
March 11th	0757	Manually scrammed Reactor because of rod insertion problem.
March 11th	2133	Reactor critical.
March 13th	1125	Turbine-Generator on line. Ramping up load.
March 14th	0200	Holding at 390 GMWE for OD-1.
March 16th	2154	Reducing power to perform rod pattern adjustment.
March 16th	2220	Stopped reducing power. Ramping back to rated power.
March 21st	0050	Start to drop load for weekly turbine test load stopped at 722 MWE.
March 21st	0116	Increasing load back to rated.

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NARRATIVE REPORT  
UNIT 1

March 23rd	2120	Reducing Reactor Power via Recirc. flow to 650 MWE to perform Turbine test & rod pattern adjustment.
March 23rd	2220	Stopped reducing power at 645 MWE. Weekly turbine test run & load to be ramped back to rated.
March 30th	2115	Reduced load for weekly turbine test & rod pattern adjustment.
March 30th	2345	Stopped reducing load at approx. 480 GMWE.
March 31st	0311	Began increasing reactor power. Ramping up to rated.

NARRATIVE REPORT  
UNIT 2

March 1st

0000

Recirc pipe replacement outage  
still in progress.

HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR MARCH 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
82-5071	10-01-82	Install heat tracing on 1" and 1.5" piping to sump in strainer pit area. Install plastic as temp. splash guards over heat tracing cable (Ref: DCR 81-107)
82-5077	10-01-82	Install heat tracing on lines and pressure switches for plant service water (Ref: DCR 82-107)
83-1919	02-15-84	Install 0-150 DC batter charger in standby D/G 1A, 1B, & 1C battery rooms/meter panels R43-P002A, B, & C. Remove 0-30 DC ammeters. (Ref: DCR 77-56)
83-6450	10-05-83	Redline and terminate cables (Ref: DCR 80-322) for HPCI sprinkler system - (MPL# 1T43).
83-7900	12-07-83	Adjust U-Bolt on support P42-CCWH-115A (Ref: LER 1981-103, DCR 81-58)
83-8008	03-05-84	Reroute conduit in HPCI room to avoid snubber struts (Ref: DCR 81-174)
83-8843	03-22-84	Terminate communication cables (Ref: cables #C91-P100-003 & R51-S002-143) for installation of VAX 11/780. Location at 112' elevation in Control Bldg. (Ref: DCR 82-172.)
84-141	01-12-84	Functional test snubber (located-HPCI El. 94'-7'SK1-2' WKL) per HNP-1-3915.
84-1076	02-28-84	Install two half-couplings & temp. switches on N2 vaporizer. (Ref. DCR 84-58).



HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR MARCH 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
84-1153	03-20-84	Replace Recirc "A" "Loss of Signal Relay" with HFA type GE 12HFA151-A9F. Located in 1B31-P003A. (Ref: UCR 82-171).
84-1192	03-05-84	Disconnect, meggar, and reterminate level instrument cables per HNP-6921 (Ref: NCR 84-35) for MPL# 1C11-N660 A,B,C,D.
84-1224	03-05-84	Replace existing press. switch for control of 1T48-F205 with new switch. (Ref: DCR 84-91).
84-1263	03-04-84	Replace core spray minimum flow bypass valve with valve from Unit 2. (Ref: DCR 84-95).
84-1333	03-07-84	Remove existing Non-Q 40 amp trip element for RPS MG Set 1K23-S004 FR2B. Adjust 50 amp trip element from U2 to 40 amp and install in U1. (Ref: DCR 84-98).

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR MARCH 1984

<u>NUMBER</u>	<u>DATE COMPLETE</u>	<u>DESCRIPTION</u>
83-2386	06-30-83	Construct an overlay weld on Elbow to Pipe Weld (2B31-1RC-12AR-G3) with ER 308L weld material on Recirc. Loop "A" Riser "G" (Ref: DCR 83-63).
83-2397	06-30-83	Construct an overlay weld on Elbow to pipe weld (2B31-1RC-12AR-H3) with ER 308L weld material on Recirc. Loop "A" Riser "H" (Ref: DCR 83-63).
83-2428	06-30-83	Construct overlay weld on Elbow to pipe weld (2B31-1RC-12AR-B3) with ER 308L weld material on Recirc. Loop "B" Riser "B" (Ref: DCR 83-63).
83-2518	06-30-83	Construct overlay weld on Manifold to Endcap weld (2B31-1RC-22BM-1) with ER 308L weld material on Recirc. Loop "B" (Ref: DCR 83-63).
83-2521	06-30-83	Construct overlay weld on elbow to pipe weld (2B31-1RC-12AR-A3) with ER 308L weld material on Recirc. "B" Riser A (Ref: DCR 83-63).
83-2532	06-30-83	Construct overlay weld on elbow to pipe weld (2B31-1RC-12AR-J3) with ER 308L weld material on Recirc. A Riser K (Ref: DCR 83-63).
83-2579	06-10-83	Construct overlay weld on manifold to endcap weld (2B31-1RC-22AM-4) with ER 308L weld material on Recirc. Loop A (Ref: DCR 83-63).

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR MARCH 1984

<u>NUMBER</u>	<u>DATE COMPLETE</u>	<u>DESCRIPTION</u>
83-2580	06-30-83	Construct overlay weld on Pipe to Elbow weld (2B31-1RC-12AR-F2) with ER 308L weld material on Recirc. Loop A, Riser F (Ref: DCR 83-63).
83-2641	06-02-83	Revise actuator orientation by rotating Rx. Bldg. Nitrogen Inerting system Isol. VLV (Ref: DCR 83-51).
83-2678	06-30-83	Construct overlay weld on Elbow to pipe weld (2B31-1RC-12AR-F-3) with ER 308L weld material on Recirc. Loop A Riser B (Ref: DCR 83-63).
83-2878	06-21-83	Construct overlay weld on Elbow to pipe weld (2B31-1RC-12BR-C-4) with ER 308L weld material on Recirc Loop B Riser C.
83-2880	06-30-83	Construct overlay weld on Pipe to Pipe Weld (2B31-1RC-12BR-E-3A) with ER 308L weld material on Recirc. Loop B Riser E (Ref: DCR 83-63).
83-2881	06-30-83	Construct overlay weld on Pipe to Elbow weld (2B31-1RC-12BR-D2) with ER 308L weld material on Recirc Loop B Riser D (Ref: DCR 83-63).
83-2882	06-30-83	Construct overlay weld on Pipe to Elbow weld (2B31-1RC-12BR-C2) with ER 308L weld material on Recirc Loop B Riser C (Ref: DCR 83-63).
83-2943	06-08-83	Increase thickness of Endcap to Manifold weld (2B31-1RC-22AM-1) per sections of HNP-2-10174 (Ref: DCR 83-63).

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR MARCH 1984

<u>NUMBER</u>	<u>DATE COMPLETE</u>	<u>DESCRIPTION</u>
83-2965	06-13-83	Increase thickness of Endcap to Manifold weld (2B31-1KC-22BM-1) per sections of HNP-2-10174 (Ref: DCR 83-63).
83-3126	01-27-84	Install cable, raceway and terminal boxes of new Radwaste Sprinkler System (Ref: DCR 82-128).
83-4788	12-14-83	Installed Engraved annunciator windows, 17 & 18, on Radwaste Sprinkler System (Ref: DCR 82-128).
83-4943	12-19-83	Calibrate level elements 2C11-LE-N060 A,B,C,&D and level switches 2C11-LS-N660 A,B,C,&D on Reactor Protection System/Scram discharge volume (Ref: DCR 82-220).
83-5809	02-29-84	Recalibrate the SRV Low Low Set Transmitters and Engineering to perform review (Ref: DCR 81-139).
83-5814	01-24-84	Adjust U-Bolt on support 2P52-531-H805 (Ref: DCR 82-257).
83-5815	01-24-84	Adjust U-Bolt on supports 2P41-55-H803, -H804, -H805, -H806, -H807 (Ref: DCR 82-257).
83-5816	01-23-84	Adjust U-Bolt on supports 2C11-CRD-H84, -H85, -H86, -H90, -H93, -H99, -H101, -H102, -H103, -H104, -H105, -H106.
83-6006	01-24-84	Adjust U-bolts on supports 2E11-RHR-H13 & -H14 (Ref: DCR 82-257).
83-6483	01-16-84	Install High Energy Line break Barrier for Air Supply valve 2C11-F009 A & B.
84-681	02-08-84	Replace the main hoist mechanical force gauge at Refueling platform (Ref: DCR 84-37).



Georgia Power Company  
Post Office Box 439  
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Georgia Power

Edwin I. Hatch Nuclear Plant

April 10, 1984  
GM-84-314

PLANT E. I. HATCH  
NRC Monthly Operating Report

Director  
Office of Inspection and Enforcement  
United States Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC  
Monthly Operating Report for Hatch Unit 1, Docket #50-321, and  
for Hatch Unit 2, Docket #50-366.

H. C. Nix  
General Manager

JAB/hh

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