

LICENSEE EVENT REPORT

CONTROL BLOCK:

1	2	3	4	5	6
---	---	---	---	---	---

 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1
---	---

1	L	Q	A	D	2
---	---	---	---	---	---

2	0	0	0	-	0	0	0	-	0	0	0
---	---	---	---	---	---	---	---	---	---	---	---

3	4	1	1	1	1
---	---	---	---	---	---

4	5
---	---

7 8 9 14 15 25 26 30 37 CAT 38CON'T

0	1
---	---

 REPORT SOURCE

L	6	0	1	5	0	0	0	2	6	5
---	---	---	---	---	---	---	---	---	---	---

 DOCKET NUMBER

7	1	0	2	8	8	3
---	---	---	---	---	---	---

 EVENT DATE

8	1	2	0	9	8	3
---	---	---	---	---	---	---

 REPORT DATE
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

On December 2, 1983, while reviewing ultrasonic test data obtained to comply with the inspection order of all large bore stainless steel piping, three linear indications were identified in the Reactor Recirculation System piping. Weld 02AS-S6, a 28 inch pipe-to-pipe weld, and Weld 02AS-S9, a 28 inch valve-to-elbow weld both on the 'A' Recirculation pump suction line, and Weld 02F-F6, a 12 inch pipe-to-pipe weld on the 'F' Recirculation riser were found to have circumferential linear indications in their heat-affected zones. The slow growth rate, typical in this type of indication, combined with the reduced allowable Containment leakage rate, would have been sufficient to readily identify any possible leakage and preclude a gross failure.

0	9
---	---

 SYSTEM CODE

C	B
---	---

 CAUSE CODE

E

 CAUSE SUBCODE

C

 COMPONENT CODE

P	I	P	E	X	X
---	---	---	---	---	---

 COMP SUBCODE

E

 VALVE SUBCODE

Z

9 10 11 12 13 18 19 20

17

 LER/RO REPORT NUMBER

8	3
---	---

 EVENT YEAR

0	2	1
---	---	---

 SEQUENTIAL REPORT NO.

0	1
---	---

 OCCURRENCE CODE

T

 REPORT TYPE

3

 REVISION NO.
21 22 23 24 26 27 28 29 30 31 32
ACTION TAKEN

X

 FUTURE ACTION

X

 EFFECT ON PLANT

Z

 SHUTDOWN METHOD

Z

 HOURS

0	0	0
---	---	---

 ATTACHMENT SUBMITTED

N

 NPSR-4 FORM SUB

N

 PRIME COMP. SUPPLIER

N

 COMPONENT MANUFACTURER

D	2	4	0
---	---	---	---

33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The cause of this occurrence is postulated as being intergranular stress corrosion cracking. Further analyses are being performed to determine the appropriate corrective action. A supplemental report will be submitted following repairs.

1	5
---	---

 FACILITY STATUS

H

 % POWER

0	0	0
---	---	---

 OTHER STATUS

NA

 METHOD OF DISCOVERY

C

 DISCOVERY DESCRIPTION

Special Inspection of SS Piping

7 8 9 10 12 13 44 45 46 80

1	6
---	---

 ACTIVITY CONTENT RELEASED OF RELEASE

Z	Z
---	---

 AMOUNT OF ACTIVITY

NA

 LOCATION OF RELEASE

NA

7 8 9 10 11 44 45 80

1	7
---	---

 PERSONNEL EXPOSURES NUMBER

0	0	0
---	---	---

 TYPE

Z

 DESCRIPTION

NA

7 8 9 11 12 13 80

1	8
---	---

 PERSONNEL INJURIES NUMBER

0	0	0
---	---	---

 DESCRIPTION

NA

7 8 9 11 12 80

1	9
---	---

 LOSS OF OR DAMAGE TO FACILITY TYPE

Z

 DESCRIPTION

NA

7 8 9 10 80

2	0
---	---

 PUBLICITY ISSUED

N

 DESCRIPTION

NA

7 8 9 10 80NAME OF PREPARER D ClarkPHONE 309-654-2241, ext 2448401050512 831209
PDR ADOCK 05000265
S PDR

IE22

NRC USE ONLY



Commonwealth Edison

Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

DMB

NJK-83-459

December 9, 1983

J. Keppler, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-265, DPR-30, Unit Two
Appendix A, Section 6.6.B.1.c

Enclosed please find Reportable Occurrence Report Number RO 83-21/01T-3 for Quad-Cities Nuclear Power Station. Revision 3 of this occurrence was previously reported to Region III, Office of Inspection and Enforcement by telephone and telecopy on December 5, 1983.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.1.c, an abnormal degradation discovered in the Reactor Coolant Pressure Boundary.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

L. J. Germer for

N. J. Kalivianakis
Station Superintendent

NJK:DGC/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center

DEC 9 1983

IEJ211