

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTROL BLOCK:

0	1	1	L	Q	A	D	2	2	0	0	0	-	0	0	0	-	0	0	0	3	4	1	1	1	1	4			5	
7	8	9	LICENSEE CODE					14	15	LICENSE NUMBER										25	26	LICENSE TYPE				30	57	CAT		58

REPORT
SOURCE

REPORT SOURCE: 01 L 6 0 5 0 0 0 2 6 5 7 1 0 2 8 8 3 8 1 2 1 3 8 3 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

On December 12, 1983, ultrasonic test data taken during inspections required by I.E.

0 3 | Bulletin 83-02 was being reviewed. One linear indication was identified on the

0	4	Residual Heat Removal (RHR) Shutdown Cooling suction piping. Weld 10S-F1, a 20 inch
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05 | tee-to-pipe weld, was found to have a circumferential linear indication in the heat-

06 | affected zone of the weld. The slow growth rate of these indications combined with

07 | the reduced allowable containment leakage rate would have been sufficient to

08 readily identify any possible leakage and preclude a gross failure.

7 8 9

0 9

7 8

SYSTEM CODE C F 11

9 10

CAUSE CODE E 12

11

CAUSE SUBCODE C 13

12

COMPONENT CODE P I P E X X 14

13 18

COMP. SUBCODE E 15

19

VALVE SUBCODE Z 16

20

(17) LER/RO REPORT NUMBER EVENT YEAR
83—
 21 22 23

SEQUENTIAL REPORT NO. 020/
 24 25 26 27

OCCURRENCE CODE 01
 28 29

REPORT TYPE T—
 30 31

REVISION NO. 2
 32

ACTION TAKEN X FUTURE ACTION X EFFECT ON PLANT Z SHUTDOWN METHOD Z HOURS 22 ATTACHMENT SUBMITTED N NRPD-4 FORM SUB. N PRIME COMP. SUPPLIER N COMPONENT MANUFACTURER D 2 4 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause of this occurrence is postulated as being intergranular stress corrosion

11 cracking. Further analyses are being performed to determine the corrective action

1 2 to be taken. A supplemental report will be submitted following corrective actions.

1 3

1 4

7 8 9
FACILITY STATUS (1) 5 (2) H (28)
% POWER (10) 0 (11) 0 (12) 0 (29)
OTHER STATUS (30) NA
METHOD OF DISCOVERY (31) C (32) I.E. Bulletin 83-02 Requirement
44 45 46 80

ACTIVITY CONTENT
RELEASED OF RELEASE

1 6 Z 33 10 34

AMOUNT OF ACTIVITY
NA

35

LOCATION OF RELEASE

36

NA

PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION
1	7	0	0	0	37 Z 38 NA

PERSONNEL INJURIES
NUMBER DESCRIPTION (41) NA
1 2 0 0 0 (40) NA
8401050485 831213
PDR ADDCK 05000265 PDR

8		9		10		11		12	
		TYPE		LOSS OF OR DAMAGE TO FACILITY DESCRIPTION					
1	9	Z	(42)	NA					

8 9 10
PUBLICATION
ISSUED DESCRIPTION (45) NA 68 69 80
2 0 N (44) NRC USE ONLY

NAME OF PREPARER D Clark

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Commonwealth Edison

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Dmb

NJK-83-471

December 13, 1983

J. Keppler, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-265, DPR-30, Unit Two
Appendix A, Section 6.6.B.1.c

Enclosed please find Reportable Occurrence Report Number R0 83-20/01T-2 for Quad-Cities Nuclear Power Station. Revision 2 of this occurrence was previously reported to Region III, Office of Inspection and Enforcement by telephone and by telecopy on December 13, 1983.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.1.c, an abnormal degradation discovered in the Reactor Coolant Pressure Boundary.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

L. J. Kervier
N. J. Kalivianakis
Station Superintendent

NJK:DGC/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center

DEC 30 1983

IE2211