

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	1	L	Q	A	D	2	2	0	0	0	-	0	0	0	-	0	0	0	3	4	1	1	1	1	4		5								
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34								
LICENSEE CODE														LICENSE NUMBER										LICENSE TYPE										CAT 58	

CON'T

01	L	6	0	5	0	0	0	2	6	5	7	1	1	2	8	8	3	8	1	2	0	5	8	3	9						
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34				
REPORT SOURCE		DOCKET NUMBER										EVENT DATE										REPORT DATE									

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On November 28, 1983, while performing Main Steam Line High Flow Calibration, Q1S

03 21-1, Pressure Switch DPIS 2-261-2K was found to trip at 148.5 PSID. This exceeds

04 the Technical Specification of 140% steam flow as established in Table 3.2-1. A

05 differential pressure of 148 PSID corresponds to a steam flow of 140%, and 148.5

06 PSID is approximately 140.2% steam flow. The three other switches on the 'C'

07 Main Steam line were operating properly. The Group 1 Isolation would have occurred

08 as designed on a steam line high flow signal. Therefore, the safety implications

09 of this occurrence are minimal.

09	C	D	11	X	12	X	13	I	N	S	T	R	U	14	S	15	Z	16	8	3	21	22	0	2	3	24	25	0	1	3	28	29	L	30		31	0	32	E	18	Z	19	Z	20	Z	21	0	0	0	0	37	38	N	23	N	24	N	25	B	0	8	0	26
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60										
SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE								COMP SUBCODE		VALVE SUBCODE		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.		ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER																			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause of this event was instrument setpoint drift. The instrument was

11 recalibrated to within limits and functionally tested satisfactorily. No

12 further corrective action is required at this time.

15	H	28	0	0	0	29	NA	30	B	31	Quarterly Calibration Surveillance	32		33	Z	34	NA	35	NA	36		37	0	0	0	38	Z	39	NA	40	0	0	0	41	NA	42	Z	43	NA	44	N	45		46		47		48		49		50	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60
FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION										ACTIVITY CONTENT RELEASED		AMOUNT OF ACTIVITY		LOCATION OF RELEASE		PERSONNEL EXPOSURES NUMBER		TYPE		DESCRIPTION		PERSONNEL INJURIES NUMBER		DESCRIPTION		LOSS OF OR DAMAGE TO FACILITY TYPE		DESCRIPTION		PUBLICITY ISSUED		DESCRIPTION													

NAME OF PREPARER P CochranPHONE 309-654-2241, ext 1818401050431 831205
PDR ADOCK 05000265
S PDR

NRC USE ONLY



Commonwealth Edison

Quad Cities Nuclear Power Station
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DMB

NJK-83-454

December 6, 1983

J. Keppler, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-265, DPR-30, Unit Two
Appendix A, Section 3.2.A, Table 3.2-1

Enclosed please find Reportable Occurrence Report Number RO 83-23/03L-0
for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of
Technical Specification 6.6.B.2.a, as an engineered safety feature
instrument setting found to be less conservative than Technical
Specifications, but which do not prevent the fulfillment of the functional
requirement of the system.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

L. J. Harner for
N. J. Kalivianakis
Station Superintendent

NJK:DGC/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center

DEC 19 1983

IEJ211