

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

SYSTEM CODE C C		CAUSE CODE X		CAUSE SUBCODE Z		COMPONENT CODE I N S T R U				COMP. SUBCODE E		VALVE SUBCODE Z	
11		12		13		14				15		16	
9 10		11 12		12 13		13 18				19 20		20	
EVENT YEAR 8 3		SEQUENTIAL REPORT NO. 0 3 9		OCCURRENCE CODE 0 3		REPORT TYPE L		REVISION NO. 0					
21 22		23 24		25 26		27 28		29 32					
ACTION TAKEN Z		FUTURE ACTION X		EFFECT ON PLANT Z		SHUTDOWN METHOD Z		HOURS 0 0 0 0					
18 19		20		21		22		23					
17		18		19		20		21					
LER/RO REPORT NUMBER		ATTACHMENT SUBMITTED Y		NPRD-4 FORM SUB. N		PRIME COMP. SUPPLIER N		COMPONENT MANUFACTURER P 0 5 0					
21 22		23 24		25 26		27 28		29 32					

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

2 0 N (44) _____ S PDR _____ 68 69

PHONE: (315) 349-2616

8401160160 840103
PDR ADCK 05000220
S PDR

NRC USE ONLY

917-926

DRAFT LER 83-39

EVENT DESCRIPTION

During normal operations while performing surveillance test N1-ISP-RVT on the main steam electromatic relief valve tailpipe temperature sensors, the inputs indicated loose connections on the thermocouple and/or failure of the thermocouple. Since the thermocouples act as backup sensors to the primary acoustic sensors which are currently operable, consequences were minimal. LER 83-15 reported a similar occurrence.

CORRECTIVE ACTIONS

The surveillance test on the sonic detector (the primary sensor) was run on valve 01-07 per N1-ISP-01-VMS and was found to be operating normally. Future corrective action to determine the cause of this event will be to investigate the thermocouple and connections upon the next scheduled Drywell entry.

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

300 ERIE BOULEVARD, WEST

SYRACUSE, N. Y. 13202

January 3, 1984

Dr. Thomas E. Murley
Regional Administrator
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

RE: Docket No. 50-220
LER 83-39

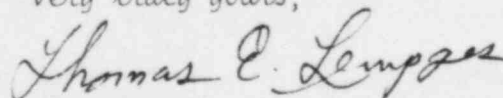
Dear Dr. Murley,

In accordance with Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit the following Licensee Event Report:

83-39 which is being submitted in accordance with Section 6.9.2.b(2), conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

This report was completed in the format designated in NUREG-0161, dated July 1977.

Very truly yours,



Thomas E. Lempges
Vice President
Nuclear Generation

TEL/jm
Attachments (3 copies)
cc: Director, Office of T&E (30 copies)
Director, Office of MIPC (3 copies)

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