

APPROVED BY OMB  
3150-0011  
EXPIRES 4-30-82

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	M D C C N 2												2	0	0	-	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5
7	8	LICENSEE CODE												14	LICENSE NUMBER										25	LICENSE TYPE					30	CAT		58

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0	1	REPORT SOURCE		L	6	0	5	0	0	0	3	1	8	7	1	2	1	7	8	3	8	1	2	2	9	8	3	9										
7	8			60	61	DOCKET NUMBER										66	EVENT DATE										74	REPORT DATE										80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | At 0850 following a reactor trip, dose equivalent I-131 was determined

0 3 | to be 1.413 micro-curies per gram (T.S. 3.4.8). At 1500, dose equivalent

0 4 | I-131 was determined to be less than 1.0 micro-curies per gram, termin-

0 5 | ating the event. Similar events: 50-317/82-20.

0 6 |

0 7 |

0 8 |

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE	
0	9	C	B	X	Z	Z	Z	Z	Z	Z	Z	Z	Z
7	8	9	10	11	12	13	14	15	16	17	18	19	20
(17) LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.			
8 3		—		0 6 8		/ 0 3		L —		0			
21 22		23		24 26		27 28 29		30 31		32			
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER					
X	Z	Z	Z	0 0 0 0	Y	N	Z	Z 9 9 9					
33	34	35	36	37 40	41	42	43	44 45 46 47					

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The depressurization shock associated with the shutdown resulted in an

1 1 "iodine spike" from the fuel. The "iodine spiking" is not an abnormal

1 2 occurrence since it is a known anticipated phenomenon. Therefore,

1 3 no additional corrective action is deemed necessary other than purifi-

1 4 cation of the system utilizing an ion exchanger.

FACILITY STATUS		% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
1	5	G	(28) 0   0   0	(29) N/A	A
					Chemistry surveillance
ACTIVITY CONTENT RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
1	6	Z	(33) Z	(34) N/A	N/A

PERSONNEL EXPOSURES

NUMBER		TYPE	DESCRIPTION
1	7	000	N/A

PERSONNEL INJURIES		NUMBER		DESCRIPTION		(41)	
1	8	0	0	0	0	0	N/A

LOSS OF OR DAMAGE TO FACILITY  
TYPE DESCRIPTION (43)

1 9 Z (42) N/A

PUBLICITY ISSUED DESCRIPTION (45) 8401160087 831229  
 PDR ADDCK 05000318  
 S PDR  
 NRC USE ONLY

NAME OF PREPARER R.J. Porter

PHONE: (301) 269-4747

Unit 2, Cycle 5 - 12/16/83 @ 1003

[illegible]

## POWER HISTORY (hourly average)

<u>DATE</u>	<u>TIME</u>	<u>POWER (MW<sub>+</sub>)</u>
12-15	0800	2695
	0900	2694
	1000	2690
	1100	2693
	1200	2695
	1300	2694
	1400	2696
	1500	2696
	1600	2694
	1700	2694
	1800	2693
	1900	2693
	2000	2692
	2100	2692
	2200	2693
	2300	2692
	2400	2693
12-16	0100	2695
	0200	2691
	0300	2695
	0400	2695
	0500	2694
	0600	2695
	0700	2695
	0800	2694
	0900	2693
	1000	2693
	1100	2693
	1200	2694
	1300	2695
	1400	2696
	1500	2697
	1600	2695
	1700	2693
	1800	2692
	1900	2698
	2000	2697
	2100	2696
	2200	2698
	2300	2594
12-17	2400	2363
	0100	2165
	0200	2144
	0300	2143
	0400	717
	0500	0
	0600	0

<u>DATE</u>	<u>TIME</u>	<u>POWER (MW<sub>t</sub>)</u>
	0700	0
	0800	0
	0900	0

Clean-up flow remained at 88 gpm starting 48 hours prior to the event.

No degassing operations took place starting 48 hours prior to the event.

# BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT  
CALVERT CLIFFS NUCLEAR POWER PLANT  
LUSBY, MARYLAND 20657

December 29, 1983

Dr. Thomas E. Murley  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region 1  
631 Park Avenue  
King of Prussia, PA 19406

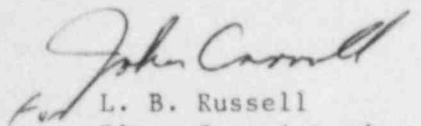
Docket No. 50-318  
License No. DPR 68

Dear Dr. Murley:

Attached is LER 83-68/3L, as required per Technical Specification 6.9.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

  
L. B. Russell  
Plant Superintendent

LBR:RJP:bsb

cc: Director, Office of Management Information  
and Program Control  
Messrs: A. E. Lundvall, Jr.  
J. A. Tiernan

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11