

NRC FORM 366
(12-81)

U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORT

APPROVED BY OMB
3150-0011
EXPIRES 4-30-82

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 MDC CN 1200-000000-000341111145
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

CON'T

01 REPORT SOURCE L6050003177710018381221839
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 At 0900 during shutdown operations while performing surveillance tests;

03 three main steam safety valves did not lift at the required setpoint.

04 RV-3995 and RV-3996 lifted prior to reaching the setpoint and RV-4006

05 lifted higher than the setpoint (T.S. 3.7.1.1). The reactor entered mode

06 5 at 2020 on 10-1 terminating the event. All other main steam safety

07 valves remained operable during the event.

08 Similar events: 50-317/82-21; 50-318/82-48.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
C C 11 X 12 Z 13 V A L V E X 14 X 15 B 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

17 LER/RO REPORT NUMBER 83
21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

18 ACTION TAKEN E 18 19 FUTURE ACTION Z 19 20 EFFECT ON PLANT Z 20 21 SHUTDOWN METHOD Z 21 22 HOURS 0000 23 ATTACHMENT SUBMITTED N 23 24 NFRD-4 FORM SUB. Y 24 25 PRIME COMP. SUPPLIER N 25 26 COMPONENT MANUFACTURER D 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The subject valves (Dresser type 3700) were disassembled, inspected,

11 and repaired during the current refueling outage. The valves were reset

12 and tested satisfactorily during plant startup. No further action is

13 deemed necessary.

14 FACILITY STATUS G 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
% POWER 000 29 N/A
OTHER STATUS 30
METHOD OF DISCOVERY C 31
DISCOVERY DESCRIPTION 32
Surveillance Tests

15 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
AMOUNT OF ACTIVITY 35
LOCATION OF RELEASE 36
N/A

16 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39
000 37 Z 38 N/A

17 PERSONNEL INJURIES NUMBER DESCRIPTION 41
000 40 N/A

18 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 43
Z 42 N/A

19 PUBLICITY ISSUED DESCRIPTION 45
N 44 N/A

20 NRC USE ONLY

NAME OF PREPARER M. A. Junge/G. S. Pavis

PHONE: (301) 269-4969/4850

8401160067 831221
PDR ADOCK 05000317
S PDR

BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

December 21, 1983

Dr. Thomas E. Murley
Regional Administrator
U. S. Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, PA 19406

Docket No. 50-317
License No. DPR 53

Dear Dr. Murley:

Attached is LER 83-55/3X, Revision 1, as required per Technical Specification 6.9.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L B Russell
L. B. Russell
Plant Superintendent

LBR:GSP:bsb

cc: Director, Office of Management Information
and Program Control
Messrs: A. E. Lundvall, Jr.
J. A. Tiernan

IE22
1/1