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OPERATING DATA REPORT

DOCKET NO. 50-321
DATE 01-10-84
COMPLETED BY: D. P. Rafeedie
TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 1
2. Reporting Period: 12-83
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 809.3
5. Design Electrical Rating (Net MWe): 777.3
6. Maximum Dependable Capacity (Gross MWe): 801.2
7. Maximum Dependable Capacity (Net MWe): 764.7
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe)
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	8759	70127
12. Number of Hours Reactor was Critical	522.2	6572.7	49525.2
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	445.0	6242.7	46393.0
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	802332	12941561	97189924
17. Gross Electrical Energy Generated (MWH)	256420	4179460	31448950
18. Net Electrical Energy Generated (MWH)	241280	3964147	29850362
19. Unit Service Factor	59.8	71.3	66.2
20. Unit Availability Factor	59.8	71.3	66.2
21. Unit Capacity Factor (Using MDC Net)	42.4	59.2	55.7
22. Unit Capacity Factor (Using DER Net)	41.7	58.2	54.8
23. Unit Forced Outage Rate	8.4	5.6	18.4
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shutdown at End of Report Period, Estimated Date of Startup:
26. Units in Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

IE 24
1/1

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-321
 DATE: 01-10-84
 COMPLETED BY: D. P. Rafeedie
 TELEPHONE (912) 367-7851

MONTH 12-83

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
-----	--

1	-6
2	-6
3	-5
4	-8
5	-6
6	-6
7	-7
8	-8
9	-13
10	-13
11	11
12	179
13	270
14	421
15	495
16	294

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
-----	--

17	97
18	447
19	529
20	653
21	707
22	719
23	678
24	590
25	681
26	742
27	213
28	304
29	678
30	735
31	690

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NARRATIVE REPORT
UNIT 1

December 1st	0000	Core reconstitution outage.
December 8th	2242	Reactor critical.
December 11th	1611	Generator tied to grid and increasing reactor power.
December 11th	2136	Manual turbine trip for overspeed testing.
December 11th	2336	Generator tied to grid increasing power to 30% thermal and holding for scram time testing to be complete.
December 13th	1038	Increasing reactor power to 50% by pulling rods.
December 13th	1235	Power increase stopped so engineer can take data.
December 14th	0200	Power ascension in progress.
December 16th	0720	Normal reactor shutdown in progress to repair "B" TIP machine.
December 16th	2054	Manual reactor scram.
December 16th	2056	Main turbine trip.
December 17th	0800	Reactor critical.
December 17th	1446	Generator tied to line and power increase started.
December 17th	1600	Power increase stopped.
December 17th	1749	Began reactor power increase by pulling rods.
December 17th	2000	Power increase stopped.
December 18th	0455	Power increase started with recirc.
December 23rd	2000	Load reduction for pattern adjustment and main turbine test.
December 23rd	2115	Weekly turbine testing in progress.

December 24th	0110	Weekly turbine test complete. Increasing reactor power.
December 27th	0657	Reactor scram due to low H ₂ O level from condensate booster pump trip.
December 27th	2321	Reactor critical.
December 28th	0553	Generator tied to grid. Power increase in progress.
December 30th	2340	Power reduction for rod pattern adjustment.
December 31st	0300	Pattern adjustment complete. Increase reactor power.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December

DOCKET NO. 50-321
 UNIT NAME Hatch 1
 DATE 1-10-84
 COMPLETED BY D.P. Rafeedie
 TELEPHONE 912-367-7851

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
83-86	831201	S	256.2	C	2	NA	RC	FUELXX	Core reconstitution outage.
83-87	831211	S	5.4	H	5	NA	RC	FUELXX	Power ascension from above outage.
83-88	831211	S	2	B	9	NA	HA	TURBINE	Manual turbine trip for overspeed testing.
83-89	831212	S	79.7	H	5	NA	RC	FUELXX	Power ascension from above outage.
83-90	831216	F	13.5	A	5	NA	ID	INSTRU	Load reduction for repair of B TIP machine.
83-91	831216	F	17.8	A	2	NA	ID	INSTRU	Outage to repair B TIP machine.
83-92	831217	S	93.2	H	5	NA	ID	INSTRU	Ramp from TIP outage.
83-93	831223	S	61	H	5	NA	RB	CONROD	Load reduction for pattern adjustment & turbine test.
83-94	831227	F	22.9	A	3	NA	CB	PUMPXX	Condensate booster pump trip.

1 F: Forced
 S: Scheduled

2 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3 Method:
 1-Manual
 2-Manual Scram.
 3-Automat. Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

4 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

5 Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December

DOCKET NO. 50-321
 UNIT NAME Hatch 1
 DATE 1-10-84
 COMPLETED BY D.P. Pafedie
 TELEPHONE 912-367-7851

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
83-95	831228	S	27	A	5	NA	CB	PUMPXX	Ramp from CBP scram.
83-96	831230	S	8.3	H	5	NA	RB	CONROD	Load reduction for pattern adjustment & main turbine weekly.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

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HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR DECEMBER 1983

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
83-7635	11-22-83	CRD Vents and Drains. Replaced existing valves C11-F110A&B (Solenoid valves) with 2 ASCO Type NP8316E36E, 125 VDC Solenoid Valves. New valves were seated as per HNP-0-6946. Ref: 83-32.
82-4426	03-28-83	Health Physics 130 El. (Smoke detection System). Installed conduits, conduit hangers, & associated electrical equipment. Pulled cable tag with opossum tails. Terminated & redline. per sketches SE 80-484-23, 24, 25, 35, 50, 56, 61, 62, 63, 68, 69, 176, 183, 185, 189, 190, and 192.
83-5714	10-06-83	HPCI Sprinkler System (Fire Protection). Installed cable, raceway, & terminal box for new HPCI sprinkler system per DCR 80-322. Installed conduits & raceways per HNP-6921, Ref: DCR 80-322.
83-6985	12-06-83	CRD SDV Vents & Drains. Removed existing supports & piping. Installed new valves C11-F035A&B, F037; piping & support per sketches on continuation sheets. Also hydrostatically tested the new installation. Ref. DCR 83-32, 82-204.
83-6950	12-05-83	Reactor Protection System/Scram Discharge Volume. Revised internal wiring in panels H11-P609 & H11-P611 & terminated cables as per attached sketches. Ref. DCR 82-219.

HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR DECEMBER 1983

82-8473	10-12-83	Turbine/Generator Bearing Fire Annunciation. Revised the annunciation logic for the 8 Unit 1 Turbine/Generator Fire alarms per sketches SE-82-134-(1,2,3,4) Rev. A & SM 82-134-1 Rev. A per DCR 82-134. Taped and stowed spare conductors. Ref. DCR 82-134.
83-8427	12-08-83	Reversed Thermocouple lead on G31-N602A-D. Ref. DCR 83-270.
83-8426	12-08-83	Leak Detection Thermocouple Selection Switch. Reversed wiring on switch B21-52 from thermocouples G31-N023A-D & G31-N022A-D per DCR 83-270. Ref. 83-270.
83-8083	12-21-83	Repaired leaks in valve C11-F035A. Ref. DCR 83-032.
83-7507	11-28-83	Reactor Protection System/Scram Discharge Volume attached new nameplates. Ref. DCR 82-219.
83-7394	12-01-83	C11-F035 Isolation Valve for the CRD Scram Discharge Volume. Disassembled Valve C11-F035 per SX-19950. Removed rust on valve parts. Reassembled valve & painted. Ref. DCR 83-032.
83-8123	11-30-83	Terminal boxes for Radiation Monitoring System. Installed TB1-1519 in West Cableway near cable tray support #9. Installed TB1-1520 in West Cableway near cable tray support #24. Installed boxes on tray TAA7-4. Ref. FDR 83-213-2, DCR 83-213.
82-7498	12-05-83	Solenoid Valves. Lubricated ASCO Solenoid Valves. Used Lubrication Kit N-220-457. Ref. DCR 79-459.

HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR DECEMBER 1983

83-6952	11-22-83	Reactor Protection System/Scram Discharge Volume. Installed level elements C11-LE-N060A,B,C,D & level switches C11-LS-N660A-D as per attached sketches & work instructions. Ref. DCR 82-219.
83-7817	11-11-83	1L11, CRD Vents & Drains, core drills for DCR 83-032.
83-6956	11-30-83	1C11 pulled 8 new cables for DCR 82-219.
83-8031	12-05-83	C11-F110A&B Install high energy line break barriers for solenoid valves. C11-F110A&B. Ref. DCR 83-032.
83-7718	12-03-83	1H11-P607 TIP monitoring & control panel implement DCR 83-21.
83-7667	11-30-83	C11-F035A&B installed conduit support & conduit. Ref. 83-032.
83-6951	12-01-83	C11 reactor protection system/scram discharge volume installed raceways & fabricated supports. Ref. DCR 82-219.
83-6964	11-18-83	1C11 Install indication lights for valves. C11-F035A&B on panel H11-P603 relocate nameplates. Ref DCR 83-032.
83-6986	12-05-83	1C11-F035A&B pulled cables for DCR 83-032.
83-7717	12-02-83	1C11-F040 Installed tubing fittings, supports, valves. C11-F045, C11-F080, C11-F081, C11-F086, supplying AIV to C11-F035A-B & C11-F037. Ref. DCR 83-032.
83-6962	12-02-83	C11-F035A&B, C11-F037, provide internal wiring for new valves & circuits. Ref. DCR 83-032.

HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR DECEMBER 1983

83-7936	12-03-83	R33 cable trays in U1 drywell restore cable trays. Ref. DCR 83-53.
83-7937	12-03-83	1R33 U1 steam chase cable trays restored. Ref. DCR 83-53.

OPERATING DATA REPORT

DOCKET NO. 50-366
 DATE 01-10-84
 COMPLETED BY: D. P. Rafeedie
 TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 2
2. Reporting Period: 12-83
3. Licensed Thermal Power (Mwt): 2436
4. Nameplate Rating (Gross MWe): 817.0
5. Design Electrical Rating (Net MWe): 784.0
6. Maximum Dependable Capacity (Gross MWe): 803.9
7. Maximum Dependable Capacity (Net MWe): 774.5
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe):
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	8760	37897
12. Number of Hours Reactor was Critical	744.0	6064.2	27215.3
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	744.0	5776.9	25932.0
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1746696	11948588	55554271
17. Gross Electrical Energy Generated (MWH)	587620	4008420	18284890
18. Net Electrical Energy Generated (MWH)	564027	3809456	17397782
19. Unit Service Factor	100.0	65.9	68.4
20. Unit Availability Factor	100.0	65.9	68.4
21. Unit Capacity Factor (Using MDC Net)	97.9	56.1	59.3
22. Unit Capacity Factor (Using DER Net)	96.7	55.5	58.6
23. Unit Forced Outage Rate	0.0	15.5	13.2
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			
RECIRC. PIPE REPLACEMENT, 1-13-84, 6 MONTHS			

25. If Shutdown at End of Report Period, Estimated Date of Startup:

26. Units in Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

DUCKET NO. 50-366
 DATE: 01-10-84
 COMPLETED BY: D. P. Rafeedie
 TELEPHONE (912) 367-7851

MONTH 12-83

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
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1	763
2	744
3	676
4	779
5	784
6	779
7	791
8	792
9	793
10	784
11	785
12	615
13	650
14	762
15	786
16	786

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
-----	--

17	764
18	640
19	771
20	785
21	791
22	791
23	791
24	789
25	784
26	780
27	785
28	780
29	780
30	788
31	614

(9/77)

NARRATIVE REPORT
UNIT 2

December 1st	1416	Load reduction due to feedwater conductivity increasing.
December 1st	1715	Increasing reactor power.
December 2nd	2110	Load reduction to fix steam leaks in condenser bay.
December 3rd	0350	Increasing reactor power with recirc.
December 10th	2209	Load reduction for weekly turbine testing.
December 11th	0307	Weekly turbine testing complete. Load increased.
December 12th	1300	Load reduction to repair leaks in steam chase.
December 13th	0821	Increasing reactor power.
December 17th	2230	Load reduction to repair feedwater line drain valve.
December 18th	0215	Increasing reactor power.
December 31st	1318	Load reduction due to 4160 "A" buss trip with a recirc pump and circ water pump trip.
December 31st	1500	Increasing reactor power.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December

DOCKET NO. 50-366
 UNIT NAME Hatch 2
 DATE 1-10-84
 COMPLETED BY D.P. Rafeedie
 TELEPHONE 912-367-7851

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
83-57	831201	F	6.7	H	5	NA	CH	ZZZZZZ	Load reduction due to feedwater conductivity increasing.
83-58	831202	F	10.9	A	5	NA	CB	HTEXCH	Load reduction to fix steam leaks in condensate bay.
83-59	831210	S	1	B	5	NA	HA	TURBIN	Load reduction for weekly turbine test.
83-60	831212	S	18	A	5	NA	HB	PIPEXX	Load reduction to fix steam leaks in steam chase.
83-61	831217	S	13.5	A	5	NA	CH	VALVEX	Load reduction to fix F/W drain valve 2N21-F154.
83-62	831231	F	10.6	A	5	NA	CB	ELECON	Load reduction due to "A" recirc pump trip from 4160 bus "A" tripping.

¹
 F: Forced
 S: Scheduled

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 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

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 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

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 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

(9/77)

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR DECEMBER 1983

<u>NUMBER</u>	<u>DATE COMPLETE</u>	<u>DESCRIPTION</u>
83-2574	05-25-83	MPL # 2C11-F090A, F092A CRD Vents & Drains. Repaired the leaking valves 2C11-F090A & 2C11-F092A. Ref. DCR 82-206.
83-8358	12-06-83	MPL # 1B31-N007A&B. Installed a new recirculation flow switch. Ref. DCR 83-254.
82-1378	05-31-83	MPL 2X45 Hot Machine Shop Flow Drain Piping Cut Out & removed concrete in floor of hot machine shop. Repoured the concrete & fitted in the necessary expansion joint. Engineering sketches B-C-79-476-1 Rev. A, B-P-79-476-22-Rev. A & P-P-79-476-18-Rev. A were used. Ref. DCR 79-476.
82-2572	09-16-83	MPL # 2P70-F103A&B Drywell Pneumatic System. Changed the setpoint of 2P70-PS-N003 from 75 psig to 92 ± 1.5 psig & changed setpoint of 2P70-F103A&B from 90 psig to 97 ± 2.0 psig. Ref. DCR 81-112, FDR 81-112-1.
82-2785	09-16-83	MPL 2P70-N017 Drywell Pneumatic System Supply Line High Pressure Switch. Calibrated 2P70-N017. Ref. DCR 81-112.
83-1967	06-15-83	Process Computer. Installed the circuit breaker 2R26-M062 in AC inverter room Unit 2 as per attached sheet. Ref. DCR 82-172.
83-2865	07-11-83	MPL 2TE-2534, states block T.B.-AA replaced states block and cable associated with it (2B21-N301A) Ref. 81-139.
83-4362	08-31-83	MPL 2P33-N054, N055 Post Accident boron and Chloride Analyzer. Installed two pressure gauges upstream & downstream of analyzers. Ref. FDR 79-476-102, DCR 79-476.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR DECEMBER 1983

83-6002	11-21-83	Diesel Generators 2A & 2C Reverse Power Relays. Changed setpoint on the ICW reverse power relays to 50 watts per attached relay data sheets. Ref. DCR 82-110.
83-2177	09-07-83	Distribution panels fabricated & placed steel numbers, reinforcing borg, & embedded conduit as shown on FDR 82-172-4. Framed & poured concrete pad for panels R25-50107 & 2R25-5132. Ref. DCR 82-172.
83-2323	11-01-83	CRD Vents & Drains. Installed indication lights for valves 2C11-F235A&B & 2C11-F037 on panel 2H11-P603. Also relocated nameplate 428 & installed nameplates 651, 652, 653, & 654 per sketches B-J-82-206-S, Rev. A & B-J-82-206-6-Rev. A. Ref. DCR 82-206.

• Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444



Georgia Power

Edwin I. Hatch Nuclear Plant

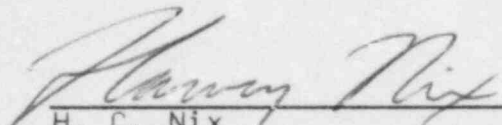
January 10, 1984
GM-84-18

PLANT E. I. HATCH
NRC Monthly Operating Report

Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC
Monthly Operating Report for Hatch Unit 1, Docket #50-321, and
for Hatch Unit 2, Docket #50-366.


H. C. Nix
General Manager

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