NRC FORM 366
(7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: | | | | | | | 1 6 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1		A	R	A	N	0	1	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5
7	8	9	LICENSEE CODE					14	15	LICENSE NUMBER											25	26	LICENSE TYPE				30	57	CAT	58

0	1	REPORT SOURCE	L	6	0	5	0	0	0	3	1	3	7	1	1	2	8	8	3	8	1	2	2	8	8	3	9
7	8	60			61	DOCKET NUMBER							68	69	EVENT DATE				74	75	REPORT DATE						80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0	2	On 11/28/83, with Unit 1 at 100% full power (FP) and Unit 2 in Mode 6, a weekly surveillance of the control
0	3	room ventilation system revealed that control room Ventilation System Fan 2VSF-9 had operated for greater than
0	4	720 hours since the last filter test. 2VSF-9 had operated for 877 hours at the time the discrepancy was
0	5	detected. Redundant Fan VSF-9 was operable. This occurrence is reportable per Unit 1 Technical Specification
0	6	(T.S.) 6.12.3.2.b and Unit 2 TS 6.9.1.9.b. A similar occurrence was reported in LER (50-313) 76-019/03L-0.
0	7	
0	8	7 8 9 80

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP SUBCODE	VALVE SUBCODE
0 9	S G 11	D 12	Z 13	F I L T E R 14	Z 15	Z 16
7 8	9 10	11	12	13 18	19	20

LER/RO	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO
17 REPORT NUMBER	8	3	0 2 7	/	0 3	L	---	0
	21	22	23	24 26	27 28	29 30	31	32

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X 18	G 19	Z 20	Z 21	0 0 0 0 22	N 23	N 24	A 25	C 0 1 0 26
33	34	35	36	37 40	41	42	43	44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1	0	The prediction from the previous week's readings did not indicate that the limit would be exceeded due to an
1	1	erroneous reading. The weekly surveillance procedure does not contain a method of data collection that has pro-
1	2	visions for cross-check of data from previous readings during the data collection. As required by TS 4.10.3.A,
1	3	a sample of carbon from the filter of 2VSF-9 was taken and sent to the laboratory for determination of iodine
1	4	removal efficiency. A procedure revision will be made to provide a method to allow cross-checking of the data.
7	8	9 80

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
1 5	E 28	1 0 0 29	Unit 2 Status = H 30	B 31	Routine Surveillance 32
7 8	9	10 12	13 44	45 46	80

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
1 6	Z 33	Z 34	NA 35	NA 36
7 8	9	10 11	44 45	80

	PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION
1 7	0 0 0 37	Z 38	NA 39
7 8	9 11	12 13	80

	PERSONNEL INJURIES NUMBER	DESCRIPTION
1 8	0 0 0 40	NA 41
7 8	9 11	12 80

	LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION
1 9	Z 42	NA 43

IE22

	PUBLICITY ISSUED	DESCRIPTION	NRC USE ONLY
2 0	N 44	NA 45	
7 8	9	10 68	69 80

NAME OF PREPARER: Patrick Rogers PHONE: (501) 964-3100

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December 28, 1983

1CAN128311

Mr. J. E. Gagliardo, Director
Division of Resident Reactor Projects
and Engineering Programs
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

Subject: Arkansas Nuclear One - Unit 1
Docket No. 50-313
License No. DPR-51
Licensee Event Report
No. 83-027/03L-0

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 1 Technical Specification 6.12.3.2.b, and Unit 2 Technical Specification 6.9.1.9.b, attached is the subject report concerning a discrepancy in the surveillance testing.

Very truly yours,

John R. Marshall

John R. Marshall
Manager, Licensing

JRM:RJS:sl

Attachment

cc: Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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