

LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

CONT

0 1 L 6 0 5 0 0 0 2 6 7 1 1 2 0 8 8 3 8 0 1 0 6 8 4 9
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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 On December 8, 1983, with the reactor shutdown, the PCRV cooling water outlet tempera-
0 3 ture exceeded the 120°F limit. This event constitutes operation in a degraded mode of
0 4 LCO 4.2.15(b) and is reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)2.
0 5 No accompanying occurrence. No effect on public health or safety. Related occurrences
0 6 are RO's 83-006, 82-054, 82-045, and 83-043.

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0 9 C B 11 A 12 A 13 Z Z Z Z Z Z 14 Z 15 Z 16
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17 LER RO REPORT NUMBER 8 3
18 ACTION TAKEN X 19 FUTURE ACTION Z 20 EFFECT ON PLANT Z 21 SHUTDOWN METHOD Z 22 HOURS 0 0 0 0
23 SEQUENTIAL REPORT NO. 0 5 2 24 OCCURRENCE CODE 0 3 25 REPORT TYPE L 26 VALVE SUBCODE Z 27 REVISION NO. 0
28 ATTACHMENT SUBMITTED Y 29 NPRD-4 FORM SUB. N 30 PRIME COMP. SUPPLIER Z 31 COMPONENT MANUFACTURER Z 9 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 With service water flow to the PCRV cooling water heat exchangers isolated, PCRV
1 1 cooling water outlet temperature increased above the 120°F limit. Service water flow
1 2 was restored to the heat exchangers, resulting in the PCRV cooling water outlet
1 3 temperature decreasing to within acceptable limits. Operations personnel were
1 4 reminded of their duty to monitor and maintain plant parameters within Technical
Specification limits. No further corrective action is anticipated or required.

1 5 G 28 0 0 0 29 N/A 30 A 31 Operator Observation
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 6 Z 33 Z 34 N/A 35 N/A 36 LOCATION OF RELEASE
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 7 0 0 0 37 Z 38 N/A 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 8 0 0 0 40 N/A 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 9 Z 42 N/A 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

2 0 N 44 N/A 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

NAME OF PREPARER

PHONE (303) 785-2224

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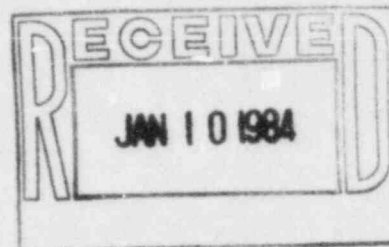


Public Service Company of Colorado

16805 Road 19 1/2, Platteville, Colorado 80651-9298

50-267

January 6, 1984
Fort St. Vrain
Unit No. 1
P-84005



Mr. John T. Collins, Regional Administrator
Region IV
Nuclear Regulatory Commission
611 Ryan Plaza Drive
Suite 1000
Arlington, Texas 76011

Reference: Facility Operating License
No. DPR-34

Docket No. 50-267

Dear Mr. Collins:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/83-052, Final, submitted per the requirements of Technical Specification AC 7.5.2(b)2.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/83-052.

Very truly yours,

Don Warembourg
Don Warembourg
Manager, Nuclear Production

DW/djm

Enclosure

cc: Director, MIPC

H005
1/1

REPORT DATE: January 6, 1984

REPORTABLE OCCURRENCE 83-052

OCCURRENCE DATE: December 8, 1983

ISSUE 0

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FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
16805 WELD COUNTY ROAD 19 1/2
PLATTEVILLE, COLORADO 80651-9298

REPORT NO. 50-267/83-052/03-L-0

Final

IDENTIFICATION OF
OCCURRENCE:

On December 8, 1983, with the reactor shutdown, the outlet temperature of the prestressed concrete reactor vessel (PCRVR) cooling water system (System 46), exceeded 120°F. This event constitutes operation in a degraded mode of LCO 4.2.15(b), and is reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)2.

EVENT
DESCRIPTION:

Following a reactor scram the morning of December 8, 1983, service water to the PCRVR cooling water heat exchangers was isolated at approximately 0840 hours. This action was taken to maintain the PCRVR cooling water outlet temperature above 100°F due to low heat loads on the system during shutdown conditions.

Decay heat present in the reactor core slowly increased the PCRVR cooling water temperature, and at approximately 0920 hours, on December 8, 1983, the outlet temperature of both cooling water loops exceeded the 120°F limit established by LCO 4.2.15(b).

An audible/visual alarm in the control room normally alerts operations personnel of rising PCRVR cooling water outlet temperature, however, this alarm was inoperable at the time. Remaining PCRVR cooling water temperature indication (TR-4637 and TR-4638) remained operable.

Upon determining high PCRVR cooling water outlet temperature, the service water supply to the System 46 heat exchangers was re-established. At approximately 1000 hours on December 8, 1983, the PCRVR cooling water outlet temperature returned to within acceptable limits.

The 120°F limit, established by LCO 4.2.15(b), was exceeded for approximately 30 minutes, with a maximum temperature of approximately 124°F.

CAUSE
DESCRIPTION:

Operator error.

Service water flow to the System 46 heat exchangers was not returned to service when PCRV cooling water outlet temperature recorders, TR-4637 and TR-4638, indicated that the outlet temperature was rising toward the 120°F limit. Absence of service water for this duration prevented the necessary heat exchange between the PCRV cooling water and the System 46 heat exchangers, resulting in a loss of the PCRV cooling water temperature control.

CORRECTIVE
ACTION:

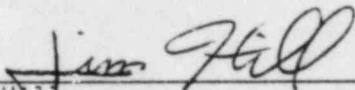
Service water flow was returned to the System 46 heat exchangers, and the PCRV cooling water outlet temperature decreased to within acceptable limits.

The audible/visual alarm was returned to service.

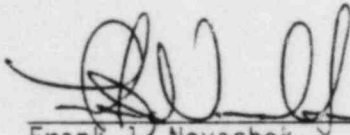
Operations personnel were reminded of their responsibility to monitor and maintain plant parameters within Technical Specification limits.

No further corrective action is anticipated or required.

Prepared By:


Jim Hill
Technical Services Technician

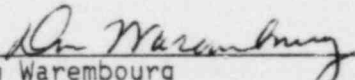
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Station Manager

Approved By:


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Manager, Nuclear Production