

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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| 7 | 8 | 9 | LICENSEE CODE | | | | | 14 | 15 | LICENSEE NUMBER | | | | | 25 | 26 | LICENSEE TYPE | | | | | 30 | 57 | 58 | |

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| 0 | 1 | L | 6 | 0 | 5 | 0 | - | 0 | 3 | 2 | 5 | 7 | 1 | 2 | 0 | 6 | 8 | 3 | 8 | 0 | 1 | 0 | 5 | 8 | 4 | 9 |
| 7 | 8 | REPORT SOURCE | | 60 | 61 | DOCKET NUMBER | | | | | 68 | 69 | EVENT DATE | | | | | 74 | 75 | REPORT DATE | | | | | 80 | |

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 While performing the operability test of the Reactor Building exhaust ventilation radi-
03 ation monitoring system, PT-04.1.1-1, it was determined that exhaust ventilation radia-
04 tion high instrument 1-D12-RM-N010B actuated at a setpoint of 13 mR/hr versus the speci-
05 fied actuation setpoint of ≤ 11 mR/hr. In accordance with Technical Specifications,
06 N010B was manually tripped to initiate a Group 6 PCIS signal which closes CAC System dry-
07 well vent/purge, inerting and containment monitoring instrumentation isolation valves.
08 This event did not affect the health and safety of the public.

Technical Specifications 6.9.1.9b

| | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
|---------------------------|---|---------------|---|----|----|-----------------|---|----|-----------------------|---|----|----------------|-----------------|----|----------------------|---------------|----|------------------|---------------|----|----------------------|----|----|------------------------|----|----|----|----|----|---|----|
| 0 | 9 | S | D | 11 | E | 12 | E | 13 | I | N | S | T | R | U | 14 | Y | 15 | Z | 16 | | | | | | | | | | | | |
| 7 | 8 | SYSTEM CODE | | 9 | 10 | CAUSE CODE | | 11 | CAUSE SUBCODE | | 12 | COMPONENT CODE | | | 13 | COMP. SUBCODE | | 19 | VALVE SUBCODE | | 20 | | | | | | | | | | |
| (17) LER/RO REPORT NUMBER | | EVENT YEAR | | 21 | 22 | SHUTDOWN METHOD | | 23 | SEQUENTIAL REPORT NO. | | 24 | 25 | OCCURRENCE CODE | | 28 | REPORT TYPE | | 30 | REVISION NO. | | 32 | | | | | | | | | | |
| ACTION TAKEN | | FUTURE ACTION | | 33 | 34 | EFFECT ON PLANT | | 35 | SHUTDOWN METHOD | | 36 | HOURS | | 37 | ATTACHMENT SUBMITTED | | 40 | NPRD-4 FORM SUB. | | 42 | PRIME COMP. SUPPLIER | | 43 | COMPONENT MANUFACTURER | | 44 | 45 | 46 | 47 | | |
| E | | Z | | 18 | 19 | Z | | 20 | Z | | 21 | 0 | | 0 | 0 | 0 | Y | | 23 | Y | | 24 | N | | 25 | G | | 0 | 8 | 0 | 26 |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 N010B actuated out of specification due to instrument setpoint drift. N010B, Part No.
11 237X892G001, was adjusted and the PT was satisfactorily completed. Performance of the
12 PT on Unit No. 2 revealed no problems. Calibration documentation associated with
13 1-D12-RM-N010B for 1981-1983 shows this instrument has a history of reliable setpoint
14 accuracy; therefore, no further action regarding this event is planned.

| | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
|---|---|-----------------|----|-------------|----|--------------------|----|-------------|---------------------|----|----|---------------------|----|---------------------|-----------------------|----|--------------------|----|----|-------------------------------|----|----|-----------|----|----|--------|----|----|----|----|----|
| 1 | 5 | E | 28 | 0 | 9 | 7 | 29 | NA | 30 | B | 31 | Periodic Test | 32 | | | | | | | | | | | | | | | | | | |
| 7 | 8 | FACILITY STATUS | | 9 | 10 | % POWER | | 11 | OTHER STATUS | | 30 | METHOD OF DISCOVERY | | 45 | DISCOVERY DESCRIPTION | | 32 | 80 | | | | | | | | | | | | | |
| 1 | 5 | Z | 33 | Z | 34 | AMOUNT OF ACTIVITY | | 35 | LOCATION OF RELEASE | | 36 | NA | | 45 | NA | | 46 | 80 | | | | | | | | | | | | | |
| 1 | 7 | 0 | 0 | 0 | 37 | Z | 38 | DESCRIPTION | | 39 | NA | | 40 | PERSONNEL EXPOSURES | | 41 | PERSONNEL INJURIES | | 42 | LOSS OF OR DAMAGE TO FACILITY | | 43 | PUBLICITY | | 44 | ISSUED | | 45 | 80 | | |
| 1 | 8 | 0 | 0 | 0 | 40 | DESCRIPTION | | 41 | NA | | 42 | NA | | 43 | NA | | 44 | NA | | 45 | NA | | 46 | NA | | 47 | NA | | 48 | 80 | |
| 1 | 9 | Z | 42 | DESCRIPTION | | 43 | NA | | 44 | NA | | 45 | NA | | 46 | NA | | 47 | NA | | 48 | NA | | 49 | NA | | 50 | NA | | 51 | 80 |
| 2 | 0 | N | 44 | DESCRIPTION | | 45 | NA | | 46 | NA | | 47 | NA | | 48 | NA | | 49 | NA | | 50 | NA | | 51 | NA | | 52 | NA | | 53 | 80 |

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PDR ADOCK 05000325
S PDR

NRC USE ONLY

NAME OF PREPARER M. J. Pastva, Jr.

PHONE: 919-457-521

LER ATTACHMENT - RO # 1-83-62

Facility: BSEP Unit No. 1

Event Date: December 6, 1983

During Unit No. 1 power operation, performance of the operability test of the Reactor Building Exhaust Ventilation Radiation Monitoring System, PT-04.1.1-1, revealed that Reactor Building exhaust ventilation radiation high instrument, 1-D12-RM-N010B, actuated at a trip setpoint of 13 mR/hr. The specified actuation trip setpoint of this instrument is ≤ 11 mR/hr.

In accordance with technical specifications, N010B was manually tripped, which initiates a Group 6 primary containment isolation signal that closes the Containment Atmosphere Control System drywell, ventilation/purge, nitrogen inerting, and containment monitoring instrumentation isolation valves. The Technical Specifications applicable to these systems are 3.3.2, 3.3.5.2, 3.3.5.3, 3.4.3.1, 3.6.6.2, and 3.6.6.4. In addition, this report is submitted in accordance with Technical Specification 6.9.1.9b.

The out-of-specification actuation of N010B is attributed to instrument drift of the N010B trip setpoint.

N010B was adjusted and the PT was satisfactorily completed. In addition, performance of the PT on Unit No.2 did not reveal any problems. During the previous performance of PT-04.1.1-1 on this instrument on November 12, 1983, N010B was left operating with a trip setpoint of 10 mR/hr. A review of the calibration documentation associated with 1-D12-RM-N010B for the last three years (1981-1983) shows this instrument has a history of reliable trip setpoint accuracy. Therefore, no further action regarding this event is planned.



Carolina Power & Light Company

JAN 10 A 9:48

Brunswick Steam Electric Plant

P. O. Box 10429

Southport, NC 28461-0429

January 5, 1984

FILE: B09-13510C

SERIAL: BSEP/84-0019

Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-83-62

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/ag/LETJ03

Enclosure

cc: Mr. R. C. DeYoung
NEC Document Control Desk

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