

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/9/ (7) /1/2/0/5/8/3/ (8) /0/1/0/4/8/4/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On December 5, 1983, with Unit 2 at 100 percent rated thermal power, the Low Head/
/0/3/ / Safety Injection Pump (1-SI-P-1A) containment sump suction valve MOV-2860A failed/
/0/4/ / to fully open during surveillance testing. Since the redundant ECCS subsystem /
/0/5/ / remained operable and the valve was returned to operable status within the time /
/0/6/ / limit of the Action Statement of T.S. 3.5.2.c, the health and safety of the /
/0/7/ / public were not affected. This event is reportable pursuant to T.S. 6.9.1.9.b. /
/0/8/ / A similar event was reported in Unit 2 LER 82-061. /

SYSTEM CAUSE CAUSE COMP. VALVE
CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE

/0/9/ /S/F/ (11) /A/ (12) /C/ (13) /V/A/L/V/E/X/ (14) /E/ (15) /D/ (16)
LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION
REPORT NO. TYPE NO.

(17) /8/3/ /-/ /0/7/2/ / / /0/3/ /L/ /-/ /0/
NUMBER

ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER

/X/ (18) /X/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /Y/ (23) /N/ (24) /N/ (25) /A/2/0/0/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / An immediate investigation revealed that an electrical cord had been hung on an /
/1/1/ / unused support attached to the valve stem extension. This prevented rotation of /
/1/2/ / the valve stem extension and would not allow the valve to fully open. The elec- /
/1/3/ / trical cord was removed and the valve was tested satisfactorily. /
/1/4/ /

FACILITY METHOD OF
STATUS %POWER OTHER STATUS (30) DISCOVERY DISCOVERY DESCRIPTION (32)
/1/5/ /E/ (28) /1/0/0/ (29) / NA / /B/ (31) / Surveillance Test /

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES
NUMBER TYPE DESCRIPTION (39)
/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES
NUMBER DESCRIPTION (41) 8401130010 840104
/1/8/ /0/0/0/ (40) / NA S PDR ADOCK 05000339 PDR

LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
/1/9/ /Z/ (42) / NA /

PUBLICITY
ISSUED DESCRIPTION (45) NRC USE ONLY
/2/0/ /N/ (44) / NA / / / / / / / / / / / /

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

Virginia Electric and Power Company
North Anna Power Station, Unit No. 2
Docket No. 50-339
Attachment to LER 83-072/03L-0

Attachment: Page 1 of 2

Description of Event

On December 5, 1983, during routine surveillance testing of the 1A LHSI pump containment sump suction valve, it was observed that the suction valve would not fully open when actuated from the Control Room. After several unsuccessful attempts to open the valve, all resulting in mid position indication, an operator was dispatched to investigate and the valve was declared inoperable. This event is reportable pursuant to T.S. 3.5.2.c and T.S. 6.9.1.9.b

Probable Consequences of Occurrence

This suction valve from the containment sump provides long term core cooling capability in the recirculation mode during the design basis accident recovery period. Each ECCS subsystem has sufficient cooling capacity for fulfilling the analyzed mitigation requirements. Since the redundant ECCS subsystem remained operable and the affected subsystem was returned to service within the time frame of the Action Statement of T.S. 3.5.2, the health and safety of the public were not affected.

Cause of Event

This valve has a 45' stem extension which connects the limitorque operator to the valve stem. An investigation revealed that an electrical cord from a temporary sump pump was coiled over an unused support on the stem extension on the second level of the Safeguards Building. This prevented the valve stem extension from rotating which caused the limitorque operator to trip on high torque. There was enough slack in the cord to allow the valve to partially open enough to clear the close limit switch. The reason the support is attached to the valve stem is unknown but it is believed that it was utilized during installation of the extension and never removed.

Immediate Corrective Action

The electrical cord was removed and the valve was tested satisfactorily.

Scheduled Corrective Action

The support which was attached the valve stem extension will be removed along with a similar support attached to the extension on the B LHSI pump suction valve.

Action Taken To Prevent Recurrence

No further action is required.

Generic Implications

A similar event occurred on September 5, 1983 and was reported in Unit 2 LER 82-061/03L-0. The Unit 1 LHSI pump containment sump suction valves were inspected and do not have this support attached to the valve stem.

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY

10 49:43
NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

January 4, 1984

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-174
NO/JRR: dus
Docket No. 50-339
License No. NPF-7

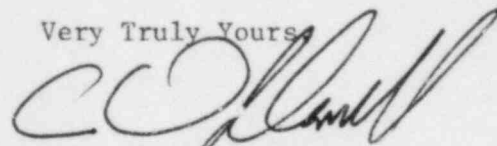
Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 2.

Report No.	Applicable Technical Specifications
LER 83-072/03L-0	T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours



E. Wayne Warrell
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
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U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

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