

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/9/ (7) /1/2/1/1/8/3/ (8) /1/2/2/1/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On December 11, 1983, with Unit 2 in Mode 1, Pressurizer Level Indication, /
/0/3/ / Channel II LI-2460 failed low. Since the remaining two channels were still oper- /
/0/4/ / able and the failed channel was placed in trip within one hour to assure conser- /
/0/5/ / vative coincidence of Reactor Protection Actuation Signals, the health and safety /
/0/6/ / of the public were not affected. This event is contrary to T.S. 3.3.1.1 and /
/0/7/ / reportable pursuant to T.S. 6.9.1.9.b. /
/0/8/ /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
/0/9/ /I/A/ (11)	/E/ (12)	/E/ (13)	/I/N/S/T/R/U/ (14)	/P/ (15)	/Z/ (16)
LER/RO REPORT NUMBER	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE	REVISION NO.

(17) /8/3/ /-/ /0/7/9/ / / /0/3/ /L/ /-/ /0/
ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER
/A/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /Y/ (24) /N/ (25) /W/1/2/0/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / The event was caused by the failure of the power supply card for LI-2460. The /
/1/1/ / card was replaced and the channel calibrated satisfactorily. No further correct- /
/1/2/ / ive action is required. There are no generic implications of this event. /
/1/3/ /
/1/4/ /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/ /E/ (28)	/1/0/0/ (29)	/ NA / (30)	/A/ (31)	/ Operator Observation /
ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)	
/1/6/ /Z/ (33)	/Z/ (34)	/ NA /	/ NA /	
PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)		
/1/7/ /0/0/0/ (37)	/Z/ (38)	/ NA /		
PERSONNEL INJURIES NUMBER	DESCRIPTION (41)			
/1/8/ /0/0/0/ (40)	/ NA /			
LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)			
/1/9/ /Z/ (42)	/ NA /			
PUBLICITY ISSUED	DESCRIPTION (45)			
/2/0/ /N/ (44)	/ NA /			

NRC USE ONLY

NAME OF PREPARER E. Wayne Harrell

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PDR ADOCK 05000339
S PDR



VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

December 21, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-172
NO/CLF: 11
Docket No. 50-339
License No. NPF-7

Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 2.

Report No.

Applicable Technical Specifications

LER 83-079/03L-0

T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,

E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
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U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

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