TENNESSEE VALLEY AUTHORITY

DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT

TO THE

NUCLEAR REGULATORY COMMISSION

NOVEMBER 1, 1983 - NOVEMBER 30, 1983

UNIT 1

DOCKET NUMBER 50-327

LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328

LICENSE NUMBER DPR-79

Submitted By: _____
Power Plant Superintendent

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Operations Summary

November, 1983

The following summary describes the significant operational activities for the month of November. In support of this summary, a chronological log of significant events is included in this report.

Unit 1

Unit 1 was critical for 567.1 hours, produced 658,150 MWH (gross), resulting in an average hourly gross load of 1,160,554 kW during the month. There are 19.6 full power days estimated remaining until the end of cycle 2 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached January 19, 1984. The capacity factor for the month was 77.3 percent. Cycle 3 refueling/modification outage is scheduled to begin February 10, 1984.

There was one reactor scram and no manual shutdowns, or power reductions during November.

Unit 2

Unit 2 was critical for 521.4 hours, produced 461,260 MWH (gross), resulting in an average hourly gross load of 903,693 kW during the month. There are 256.6 full power days estimated remaining until the end of cycle 2 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached September 27, 1984. The capacity factor for the month was 54.2 percent.

There were two reactor scrams (unit not on-line), two manual shutdowns, and two power reductions during November.

Significant Operational Events

Unit 1

Date	Time	Event
11/01/83	0001	Reactor in mode 1 at 100% power producing 1160 MWe.
11/24/83	1503	The reactor tripped after the turbogenerator tripped when an electrical fault alarm was received on the main generator.
11/25/83	1537	The reactor entered mode 4.
11/28/83	0958	The reactor entered mode 5.
11/30/83	2359	The reactor is in mode 5. The investigation continues to determine the cause of the electrical fault in the main generator.

Significant Operational Events

(Continued)

Unit 2

Date	Time	Event
11/01/83	0001	The reactor was in mode 1 at 100% power producing 1185 MWe.
	2230	Began load reduction due to high vibrations on reactor coolant pump #1.
11/02/83	0128	Unit taken off-line.
	0147	Reactor entered mode 3.
	1042	Reactor entered mode 4.
11/03/83	0038	Reactor entered mode 5.
11/06/83	0926	Began unit heat-up.
11/07/83	0424	Reactor entered mode 4.
	1137	Reactor entered mode 3.
11/09/83	1237	Reactor taken critical.
	1330	Reactor entered mode 1. The unit is not tied on-line.
	1335	Reactor power held at 9% due to steam generator chemistry.
	1551	Unit tied on-line. Increasing power to 30%.
	1910	Holding reactor power at 30%, producing 280 MWe due to steam generator chemistry
11/13/83	1628	Began power ascension.
	2200	Held reactor power at 75% while placing the cooling towers in service.
11/14/83	0400	Began power ascension.
	0930	Held reactor power at 87% while adjusting feedwater controls.
	1000	Began power ascension
	1430	Reactor at 100% power producing 1183 MWe.

Significant Operational Events

(Continued)

Unit 2

Date	Time	Event
11/15/83	0743	Began reducing load due to control oil problems on both main feed pumps.
	0752	Held load at 96% due to control oil problems on the MFP's.
	0827	Began power ascension.
	0900	Reactor at 100% power producing 1186 MWe.
	1057	Began load reduction due to control oil problems on the MFP's.
	1119	The reactor was at 85% power and holding. This was the maximum power the MFP's could handle.
	1121	The MFP's oil strainers were found clogged.
	1155	Began load reduction.
	1250	Held reactor at 48% to work on the MFP's oil strainers.
11/16/83	0525	Began power ascension.
	1400	The reactor obtained 100% power producing 1183 MWe.
11/18/83	1800	Began reducing load due to high vibrations on reactor coolant pumps #1 and #4.
	2113	Unit taken off-line.
11/18/83	2140	Reactor entered mode 3.
11/19/83	1142	Reactor taken critical
	1330	Reactor entered mode 1 (unit not on line).
	1456	The reactor was at 20% power and the unit not tied on-line when the reactor tripped due to feedwater/ steam flow mismatch during startup.

Significant Operational Events
(Continued)

Unit 2

Date	Time	Event
	1720	Reactor taken critical.
	1920	The reactor was at 11% power and the unit was not on-line when the reactor tripped due to feedwater/steam flow mismatch during start-up.
	2245	Reactor taken critical.
11/20/83	0030	The unit tied on-line.
	0055	Reactor holding power at 30% due to secondary water chemistry.
	2027	Began power ascension.
11/21/83	1400	Reactor power 100% power producing 1181 MWe.
	0855	Began load reduction due to injection water problems on the MFP's.
	1400	Held reactor power at 50% for maintenance on MFP's.
11/26/83	0630	Began power ascension.
	1120	Reactor at 100% power producing 1183 MWe.
11/30/83	2359	Reactor at 100% power producing 1183 MWe.

Fuel Performance

Unit 1

The core average fuel exposure accumulated during November was 907.79 MWD/MTU with the total accumulated core average fuel exposure of 10412.82 MWD/MTU.

The first fuel shipment (cycle 3) arrived on November 9. A total of 48 fuel bundles were satisfactorily inspected, processed and stored in the new fuel storage vault. Twelve fuel bundles received on the first shipment have incorrect ANSI numbers. It is planned that two Westinghouse representative from Columbia, S.C. will be on-site on or about the first of January to correct the discrepancy.

Unit 2

The core average fuel exposure accumulated during November was 647.21 MWD/MTU with the total accumulated core average fuel exposure of 1094.10 MWD/MTU.

PORV'S and Safety Valves Summary

No PORV's or safety valves were challenged during the month.

Licensee Events and Special Reports

The following Licensee Event Reports (LER's) were sent during November 1983, to the Assistant Director of Nuclear Power (Operations) for reporting to the Nuclear Regulatory Commission.

Unit 1

LER	SUBJECT
SQRO-50-327/83135	During recalibration of unit 2 reactor trip instrumentation for overtemperature delta T, it was found that the calibration procedure contained incorrect valves. A review of unit 1's calibration data indicated that all four unit 1 overtemperature delta T channels setpoints were less conservative than Tech Spec allowable limits.
SQRO-50-327/83138	Containment isolation valve 1-FCV-43-64 was discovered leaking.
SQRO-50-327/83139	Primary containment internal pressure exceeded +.3 psig relative to the annulus. Maximum pressure reached approximately .415 psig and occured while purging containment. It was discovered that exhaust fan suction damper 1-FCV-30-61 was not opened when the system was lined-up.
SQRO-50-327/83142	On October 7, 1983, with unit 1 in mode 1 and unit 2 in mode 3, during a review of modifications involving safety related snubbers, it was discovered that the plant procedure listing snubbers did not agree in number with table 4.7.9A of LCO 3.7.9.
SQRO-50-327/83143	During a review of SI's it was discovered that "H" WGDT had a high oxygen concentration of 2.2% on October 6, 1983 and "A" WGDT had a high oxgyen concentration of 2.3% on October 22, 1983.
SQRO-50-327/83150	On October 25, 1983 one channel of the reactor coolant pump underfrequency trip system was declared inoperable due to failure to meet surveillance requirements.
SQRO-50-327/83152	Reactor coolant drain tank nitrogen supply line containment isolation valve 1-FCV-77-20 was found inoperable during surveillance testing on October 28, 1983.
SQRO-50-327/83154	"J" WGDT had a high oxygen concentration of 3.5% and "H" WGDT had a high oxygen concentration of 2.35% on November 6, 1983.

Licensee Events and Special Reports

(Continued)

Unit 1

LER	SUBJECT
SQRO-50-327/83155	1-FT-3-103A loop 4 FW flow transmitter was delcared inoperable on November 4, 1983 when the test on line isolation valve pulled loose from the line while instrument mechanics were connecting test equipment.
SQRO-50-327/83156	UHI level switch 1-LS-87-23 was declared inoperable on November 1, 1983 when it failed to meet surveillance requirements.
SQRO-50-327/83158	The reactor coolant system subcooling margin monitor for accident monitoring instrumentation was declared inoperable on November 8, 1983 due to the failure of the plant process computer.
SQRO-50-327/83159	"D" WGDT had a high oxygen concentration of 2.3% on November 10, 1983. "A" WGDT had a high oxygen concentration of 2.1% on November 11, 1983.

Unit 2

LER	SUBJECT
SQRO-50-328/83140	Containment sump level channels 2-LT-63-176, -177, and -178 were found out of calibration during the performance of SI-202 on October 3, 1983.
SQRO-50-328/83141	Steam header pressure channel 2-PS-1-9 was found out of calibration.
SQRO-50-328/83144	During the performance of the 18-month calibration of the reactor trip system instrumentation, RCS flow channel 2-FT-68-6B and the power range flux tilt overtemperature bistable were found out of tolerance.
SQRO-50-328/83145	Steam generator level channels 3-LT-3-39 and -107 were found out of tolerance.
SQRO-50-328/83146	Pressurizer level channels 2-LT-66-320 and -335 were found out of calibration on October 12, 1983.
SQRO-50-328/83147	Steam generator level channel 2-LT-3-106 failed to meet surveillance requirement on October 9, 1983 and was declared inoperable.
SQRO-50-328/83148	The condenser vacuum exhaust flow rate monitor failed high and was declared inoperable on October 22, 1983.

Licensee Events and Special Reports

(Continued)

Unit 2

LER	SUBJECT
SQRO-50-328/83149	On November 9, 1983 pressurizer PORV 2-FCV-68-340 was found leaking through.
SQRO-50-328/83153	On October 31, 1983 engineering personnel found the outer door of the upper personned airlock not fully closed.
SQRO-50-328/83157	During the review of response time testing of the RCS flow loop temperature channels, on November 2, 1983, it was found that protection channel I 2-TE-68-2A and -14A for delta T/Tavg were not completed within the required time frame.
SQRO-50-328/83160	The turbine driven AFW pump failed to obtain 880 gpm minimum flowrate during the performance of SI-166.32 on November 2, 1983.
SQRO-50-328/83161	The rod bottom light did not come on at shutdown for control rod H-10 on November 2, 1983 and was declared inoperable.

Special Reports

There were no special reports transmitted during the month of November.

Offsite Dose Calculation Manual Changes

There were no changes to the Sequoyah Nuclear Plant ODCM during the month.

OPERATING DATA REPORT

DOCKET NO. 50-327
DATE DECEMBER 7 1983
COMPLETED BY M. G. EDDINGS
TELEPHONE (615) 870-6248

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1 NOTES:
2. REPORT PERIOD: NOVEMBER 1 - 30 1983
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS 3 THROUGH 7)SINCE LAST REPORT, GIVE REASONS:
9. POWER LEVEL TO WHICH RESTRICTED,IF ANY(NET MWE):
10. REASONS FOR RESTRICTIONS,IF ANY:

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.00	8016.00	21193.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	567.05	6905.86	14441.56
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	567.05	6793.65	14113.15
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	1929870.12	22209108.30	45491850.30
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	658150.00	7621600.00	15379136.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	632305.00	7340924.00	14776928.00
19. UNIT SERVICE FACTOR	78.76	84.75	66.59
20. UNIT AVAILABILITY FACTOR	78.76	84.75	66.59
21. UNIT CAPACITY FACTOR(USING MDC NET)	76.50	79.77	60.74
22. UNIT CAPACITY FACTOR(USING DER NET)	76.50	79.77	60.74
23. UNIT FORCED OUTAGE RATE	21.24	10.04	14.24

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
February 1984, Cycle 2 Refueling 60+ days.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
December 28, 1983

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327
UNIT NAME Sequoyah One
DATE December 8, 1983
COMPLETED BY M. G. Eddings
TELEPHONE (615)870-6248

REPORT MONTH NOVEMBER, 1983

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
26	831124	F	152.95	A	3				Reactor/Turbine Trip Electrical Fault Alarm On Main Generator.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing Outage
5-Reduction
9-Other

4
Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I-Same Source

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-327
UNIT One
DATE December 8, 1983
COMPLETED BY M. Eddings
TELEPHONE (615)870-6248

MONTH NOVEMBER, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1118	17	1133
2	1115	18	1133
3	1106	19	1130
4	1110	20	1133
5	1112	21	1128
6	1113	22	1129
7	1114	23	1128
8	946	24	705
9	1109	25	0
10	946	26	0
11	1130	27	0
12	1129	28	0
13	1132	29	0
14	1131	30	0
15	1083	31	
16	1130		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-328
DATE DECEMBER 7,1983
COMPLETED BY D.C.DUPREE
TELEPHONE (615)870-6543

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2 NOTES:
2. REPORT PERIOD: NOVEMBER 1-30,1983
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS 3 THROUGH 7)SINCE LAST REPORT, GIVE REASONS:_____
9. POWER LEVEL TO WHICH RESTRICTED,IF ANY(NET MWE):_____
10. REASONS FOR RESTRICTIONS,IF ANY:_____

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.00	8016.00	13153.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	521.40	5728.37	9617.07
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	510.42	5603.57	9410.32
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	1375955.46	17727818.69	29882409.49
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	461260.00	6070130.00	10151980.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	440100.00	5839885.00	9766175.60
19. UNIT SERVICE FACTOR	70.89	69.90	71.55
20. UNIT AVAILABILITY FACTOR	70.89	69.90	71.55
21. UNIT CAPACITY FACTOR(USING MDC NET)	53.24	63.46	64.68
22. UNIT CAPACITY FACTOR(USING DER NET)	53.24	63.46	64.68
23. UNIT FORCED OUTAGE RATE	29.11	4.50	9.92

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

NOTE THAT THE THE YR.-TO-DATE AND
CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-328
UNIT NAME Sequoyah Two
DATE December 8, 1983
COMPLETED BY D. C. Dupree
TELEPHONE (615)870-6543

REPORT MONTH NOVEMBER, 1983

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method Of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
15	831102	F	182.38	B	1				High Vibration on #1 R.C.P.
16	831115	F	0	B	5				Maintenance on the Control Oil System (Main Feed Pumps).
17	831118	F	27.20	B	1				High Vibration on #1 & #4 R.C.P.
18	831125	F	0	B	5				Repaired the Injection Water Supply Line to "B" M.F.P.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing Outage
5-Reduction
9-Other

4
Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I-Same Source

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-328
UNIT Two
DATE December 8, 1983
COMPLETED BY D. C. Dupree
TELEPHONE (615)870-6543

MONTH NOVEMBER, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1125	17	1147
2	0	18	957
3	0	19	0
4	0	20	275
5	0	21	1023
6	0	22	1152
7	0	23	1151
8	0	24	1150
9	73	25	882
10	280	26	970
11	141	27	1149
12	251	28	1144
13	250	29	1145
14	1040	30	1147
15	839	31	
16	903		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

Plant Maintenance Summary

The following significant maintenance items were completed during the month of November 1983:

MECHANICAL MAINTENANCE

1. Replaced condenser vacuum pump 1B.
2. Plugged B spent fuel pit heat exchanger.
3. Welded a scab plate on the ERCW line to the CCS heat exchanger.
4. Realigned number 1 reactor coolant pump (unit 2).
5. Furmanite repaired the unit 2 number 3 thermal barrier test flange.
6. Balanced the unit 2 reactor coolant pumps number 1 and 4.
7. Cleaned the control oil tanks and orifices on unit 2 MFPT's.
8. Removed the bladder from the monitor tank.
9. Repaired the unit 1 steam dump valves.
10. Began inspecting the MSR's on unit 1.
11. Began the ice basket servicing on unit 1.
12. Repaired the steam leaks on unit 1 MFPT's.
13. Inspected and repaired both units fuel transfer wafer valves and the unit 1 upender.
14. Replaced some HP heater valves using freeze plugs for isolation.

Electrical Maintenance

1. Checked the oil levels on unit 1 reactor coolant pump motors. Added oil to the 1, 2, & 4 motor lower bearings.
2. Completed the walkdown inspection of the E-Field wiring and made necessary repairs.
3. Returned the intertie transformer bank to its original configuration after repairs to all tap changers and diverter switches.
4. Repaired the heat trace on various boric acid lines to allow better operation of the equipment.
5. Replaced defective cells on 1B-B and 2B-B D.G. battery banks due to low specific gravity.
6. Adjusted the unit 1 pressurizer heater group controlls to allow for more efficient operation.
7. Located the grounded bar in the unit 1 generator stator.

Instrument Maintenance

Unit 1

1. Performed monthly testing of the unit 1 and 2 UHI accumulator level swithces. All switches were found within Tech Spec tolerance and three were found within desired tolerance. Two switches on unit 2 were found with wiring errors. This was reported by SQRO-50-327/83151.

2. Started working the refueling outage surveillance tests and response time testing when unit 1 tripped due to the generator fault.

3. Steam flow transmitter FT-1-28B failed high when unit 1 tripped. Inspection of transmitter revealed that apparently oil had leaked from the torque tube. We immediately shipped the transmitter to Barton for repair.

4. Reprogrammed the P-250 computer to correct the totalized feedwater flow rate input due to nozzle fouling to increase the accuracy of the calculated value of reactor thermal power.

5. Replaced a universal logic card in Train B of SSPS which was found bad during monthly functional testing.

Unit 2

1. Replaced the circuit board in PRT level transmitter.

2. Response time tested 12 narrow range RTDs on the Reactor Coolant System.

3. Change out the control oil, cleaned the orfices, and calibrated the EH converter on A and B MFPT controls during the forced outage.

4. RPI channels M2 and B10 were erratic indicating high resistance in the connectors. Reliable contact was achieved by applying voltage to the field cable and connectors.

5. Checked the stroke and response of loops 2, 3, and 4 feedwater reg valves.

INSTRUMENT MAINTENANCE SUMMARY
FOR THE MONTH OF NOVEMBER 1983

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
11/17	92	U-2 XX-92-5020	Recal Hi Lvl Trip & Rod Stop Bistables.	Recal For Safe Operation.	New Core New Data From Nuc Engr.	None	Recal
11/17	92	U-2 XX-92-5026	Recal Hi Lvl Trip & Rod Stop Bistables.	Recal For Safe Operation.	New Data From Nuc Engr.	None	Recal
11/17	68	2-LT-68-312C	Check out level Xmtr.	None	Out of Cal.	None	Recal
11/17	99	2-SSPS-99-K615	Relay energized but didn't latch.		Bad relay	None	Replaced Relay & check per SI-26.2B.
11/17	99	2-SSPS-99-K615	Relay didn't latch.		Bad relay	None	Replaced Relay & check per SI-26.2B.
11/17	63	2-LI-63-109	Channel fails out the top.	None	Strain gage bad in xmtr.	None	Replaced Strain Gage & recalled xmtr.
11/17	90	1-RM-90-106B	Will not initiate contmt. Vent isol.	None	Relay out of socket.	None	Reinstalled Relay & Ran SI-82.
11/17	68	2-LT-68-300	Suspect Inl xmtr has failed.		Bad circuit board.	None	Replaced circuit board & recalled xmtr.

INSTRUMENT MAINTENANCE SUMMARY
FOR THE MONTH OF NOVEMBER 1983

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
11/17	82	0-PS-82-271	Will not maintain 250-300 psi pressure.	None	Dirty contacts calibration.	Low Diesel starting press.	Cleaned contact, recal.
11/17	3	1-FT-3-103A	Fittings (verified installation).	None	PM on fittings.	Vlv blew off & burned IM.	Tightened fittings & verified installation.
11/17	3	1-FT-3-48	Verify valve & fitting installation).	None	PM on fittings.	Vlv blew off & burned IM.	Tightened fittings & verified installation.
11/17	3	1-FE-3-48	Verify proper installation of valves.	None	PM on valve & fittings installation.	Vlv blew off & burned IM.	Tightened fittings & verified installation.
11/17	3	1-FE-3-90	Verify proper installation of valves.	None	PM on valve & fittings installation.	Vlv blew off & burned IM.	Tightened fittings & verified installation.
11/17	68	2-PT-68-323	Check Cal of xmtr.	None	Out of calibration	None	Recal xmtr.

Field Services Group

1. ECNs 2780/5200--Post-Accident Sampling Facility (Units 1 and 2)

Conduit and cable installation is complete in the unit 1 annulus and inside the unit 1 primary containment for the post accident sampling system. Work is continuing to install cable in the unit 2 annulus. Cable is being pulled for the PASF lighting and is nearing completion. Work continued on the installation of duct for the HVAC systems serving the PASF. Installation of PASF area conduit and electrical components as well as sample tubing outside containment continues. Rework of the staircase leading from the PASF area continued. Prefabrication of hangers has begun for the PASF area breathing station air system.

2. ECN 5743--Pressurizer Enclosure Access Platform (Units 1 and 2)

Work is underway on the unit 1 platform during the current forced outage.

3. ECN 5370--Replacement of Class IE Electrical Motors

Speed readings were obtained on various motors to be replaced in the plant.

4. ECN 5106--Reactor Pressure Vessel Level Indication System

Effort is underway to begin tubing hanger installation inside the primary containment during the present unit 1 forced outage. Conduit installation inside the unit 1 primary containment is to begin during the unit 1 forced outage as well. The majority of conduit and cable work outside of containment has been completed. No work has begun in the main control panels.

5. ECN 5009--ERCW Piping Changeout (Units 1 and 2)

Work was completed to changeout ERCW piping serving the 1A1, 1A2 and 1A3 penetration room coolers. A portion of the ERCW piping, serving the 1A pipe chase cooler was replaced. Prefabrication of stainless steel piping began this month for replacement of ERCW piping serving the unit 1 spent fuel pit pump and thermal barrier booster pump area space cooler. Work has begun during the current unit 1 forced outage to change out ERCW pipe serving the safety injection system pump 1B oil cooler and room cooler as well as for the centrifugal charging pump 1A oil cooler and room cooler.

6. ECN 5645--Steam Generator Blowdown (SGBD) System (Units 1 and 2)

Nonoutage mechanical work is continuing for both units as work concentrated on installing unit 1 instrument panels, tubing, and hangers. Prefabrication of piping for unit 2 heat exchanger tie-ins and miscellaneous unit 1 hanger and insulation work continued. Work began to repair the unit 1 isolation valve 1-VLV-15-868 serving ventilation piping on the unit SGBD pump casings. Unit 1 conduit installation has begun as well.

Field Services (continued)

7. ECN 5198--Technical Support Cneter (TSC) (Units 1 and 2)

The hardware portion of the site acceptance test is complete. Work is now complete to install the TSC power distribution equipment and associated conduit on elevation 669 in the control building. The unit 1 SMS cabinet modification is complete. Installation of the TSC power and manufacturer output cables lack only obtaining the final configuration of cables which are routed to the old TSC area. Installation of TSC conduit is continuing throughout the control building. Installation of conduit serving the unit 1 main steam blowdown valves is complete. Layout of conduit routing inside the unit 1 primary containment has begun during the present forced outage as well as conduit installation in the unit 1 valve rooms.

8. ECN 5847--Plant Fire Dampers (Units 1 and 2)

All of the 89 plant fire dampers have been modified. Eight of the 15 plant fire dampers to be replaced on this ECN have been completed.

9. ECN 5647--Main Feed Pump Turbine (MFPT) Condenser Air Removal Piping (Units 1 and 2)

Prefabrication of pipe and pipe supports is continuing for unit 1. The unit 1 piping is scheduled to be replaced during the current unit 1 forced outage.

10. ECN 6695--Water Treatment System Neutralization

Work was completed this month to install control air tubing for the system. All mechanical work is complete for this ECN.

11. ECN 5237--Laundry Facility

Field work continued this month on the laundry facility to install HVAC ductwork. Work continued this month to install the new chiller package in the service building on elevation 706. Installation of conduit to the exhaust and supply fans is in progress on elevation 718. Cables are being pulled for the fire protection equipment.

12. ECN 5644--Hotwell Makeup/Dumpback Piping

The piping is now being installed during the current forced outage from CST "A" pipe isolation valve to the condenser hotwell for unit 1. This piping is being tied into the condenser in order to make this operational by the end of the current unit 1 forced outage. Piping has been installed but has not yet tied-in on unit 2. Heat trace was placed on outdoor piping serving CST "A" this month.

13. L1902--Add Isolation Valve and Trap to Pipe Serving Main Feedpump Turbine Drain Tank

This work was accomplished for unit 2. Preparations are being made to install these piping components during the current unit 1 forced outage.

Field Services (continued)

14. ECN 5664--Wells Fargo Tamper Indication

Prefabrication of electrical component assemblies to be installed for this modification is continuing as well as checkout of existing cables.

15. LDCR 1883--Liquid N_2 System Suppoort Facilities

Remaining work was completed this month on the access platform.

16. ECN 2974--Complete Response Time Cables-Unit 2 Refueling Water Storage Tank (RWST)

Field work was completed this month to install unit 2 RWST level transmitter response time testing cables.

17. ECN 5495--Field Services Building Electrical Work

Work is continuing to complete the building electrically as material and manpower allow.

18. ECN 5939--Main Feed Pump Turbine Condensor (MFPTC)

Installation of temporary platform is in progress this month in preparation for the feedwater heater tube bundle replacement during the unit 1 cycle 2 refueling outage.

19. DCR 935--Spent Fuel Pit Bridge Crane Load Cell

Work was completed this month for this DCR to replace the spent fuel pit bridge crane hoist load cell.

20. ECN 5560--Manhole Fire Detector Removal

Work began this month to remove manhole fire detectors.

21. ECN 5903--Strainers and Access Platform

Strainers are being installed upstream of the laundry facility and hot shower drain tanks. The access platform has been completely installed.

22. ECN 5194--Gaseous Effluent Radiation Monitoring

Conduit installation has begun for this modification. Core drilling of the shield building wall for tubing installation is scheduled to begin during the current forced outage.

23. ECN 5457--Replacement of Solenoid Valves

Work has resumed to change out solenoid valve parts prior to replacing existing solenoid valves. In addition, the unit 1 train "A" ERCW solenoid valves have been replaced.

Field Services (continued)

24. ECN 6031--Safety Injection System Hanger Modification

A hanger was installed on the operator for 2-VLV-63-615. Other hanger documentation will be done on this ECN.

25. ECN 5867 Fuel Transfer System

Work resumed at an accelerated pace as it was decided that the unit 1 system was to be operational by the end of the current unit 1 forced outage.

26. ECNs 5119 & 5871--Relocate ERCW Radiation Monitors 0-RE-90-133/140 & 0-RE-90-134/141

Field work has begun to reroute some conduit serving these monitors that were installed per ECN 5119. Electrical work per ECN 5871 will follow that for ECN 5119.

27. ECN 5723--Condenser Sparger for Condensate System

The unit 1 sparger is being replaced during the present forced outage by a larger diameter one which is to be installed in a new configuration.

28. ECN 6047 Reactor Coolant System Level Indication

Preparations are being made to install the level tubing during the current unit 1 forced outage.

29. ECN 5770--Condenser Vacuum Exhaust High Range Noble Gas Effluent Radiation Monitors

Work progressed throughout November for the unit 1 portion of this ECN as the radiation monitor was set and conduit was installed.

30. ECN 6050--High Pressure Fire Protection System Modification

An isolation valve has been installed in the portion of this sytem serving the control building mechanical equipment room. The system pipe supports are being modified as a result of the valve installation.

31. ECN 5713--Steam Generator Upper Manway Access Platform

Preparations are in progress to install the steam generator No. 3 upper manway access platform during the current unit 1 forced outage.

32. ECN 5034--Steam Generator Access Platforms/Ladders

Prefabrication is underway for a ladder to be installed from the elevation 679 floor in order to provide access to 1 and 4 steam generators.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
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Soddy-Daisy, Tennessee 37379

DEC 15 1983

Nuclear Regulatory Commission
Office of Management Information
and Program Control
Washington, DC 20555

Gentlemen:

Enclosed is the November 1983 Monthly Operating Report to the NRC for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

C. C. Mason
Power Plant Superintendent

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