

CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)01 C A S O S 3 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 50

CONT

01 REPORT SOURCE L 6 0 5 0 0 0 3 6 2 7 1 1 1 9 8 3 8 1 2 1 9 8 3 9  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
DOCKET NUMBER EVENT DATE REPORT DATE

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On 11/19/83, at 0213, with Unit 3 in Mode 1, both main feedwater pumps

03 tripped due to low suction pressure. The operator manually tripped the

04 reactor and actuated the Auxiliary Feedwater (AFW) system. During plant

05 stabilization Condensate Storage Tank (CST) 3T-121's level dropped

06 below the limit of LCO 3.7.1.3, and the associated Action Statement was

07 invoked. There was no impact on the health and safety of plant person-

08 nel or the public associated with this event.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE  
W F 11 X 12 Z 13 A C C U M U 14 Z 15 Z 16

17 LER-80 REPORT NUMBER 18 3 19 1 20 3 21 0 22 3 23 0 24 3 25 0 26 3 27 0 28 3 29 0 30 3 31 0 32 3 33 0 34 3 35 0 36 3 37 0 38 3 39 0 40 3 41 0 42 3 43 0 44 3 45 0 46 3 47 0 48 3 49 0 50 3

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB PRIME COMP SUPPLIER COMPONENT MANUFACTURER (26)

X 18 X 19 Z 20 Z 21 0 0 0 0 0 N 23 N 24 A 25 B 4 7 0

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause was the high water demand placed on 3T-121 by the AFW system.

11 3T-121's level was restored on 11/19/83 at 0405. A review of the auto-

12 matic level control system for the CST and a proposed Technical Specifi-

13 cation change on its minimum level requirements are in progress. (See

14 also LER 83-078, Docket No. 50-361.)

15 FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)  
B 28 0 0 0 29 NA A 31 Operator Observation

16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)  
Z 33 Z 34 NA NA

17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)  
0 0 0 37 Z 38 NA

18 PERSONNEL INJURIES NUMBER DESCRIPTION (41)  
0 0 0 40 NA

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)  
Z 42 NA

20 PUBLICITY ISSUED DESCRIPTION (45)  
N 44 NA

NAME OF PREPARER

J. G. HAYNES

PHONE

714/492-7700

8401040360 831219  
PDR ADOCK 05000362  
S PDR

RECEIVED  
NRC

*Southern California Edison Company*

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

1983 DEC 23 AM 10:37

SCE

REGION V

TELEPHONE  
(714) 492-7700

J. G. HAYNES  
STATION MANAGER

December 19, 1983

U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region V  
1450 Maria Lane, Suite 210  
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-362  
30-Day Report  
Licensee Event Report No. 83-103  
San Onofre Nuclear Generating Station, Unit 3

Pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written report and a copy of the Licensee Event Report (LER) form for an occurrence involving Limiting Condition for Operation (LCO) 3.7.1.3, associated with Condensate Storage Tanks (CST's).

On November 19, 1983, at 0150 with Unit 3 in Mode 1 at 90% power, the operator was notified of a saltwater leak in the southwest hotwell, and a ramp down in power was commenced at 0153. At 0213, after both main feedwater pumps tripped on low suction pressure, the operator manually tripped the reactor and actuated the Auxiliary Feedwater (AFW) system to provide feedwater flow to the steam generators. During stabilization of plant conditions following the trip, CST 3T-121's level dropped below 144,000 gallons, contrary to LCO 3.7.1.3. The associated Action Statement was satisfied when tank level was restored at 0405.

The low level condition in CST 3T-121 was caused by the high water demand placed on it by the AFW system. This demand temporarily exceeded the make-up flowrate of the automatic level control system for 3T-121 during steam generator level stabilization.

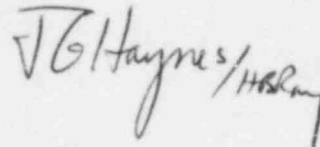
11 IE-22

December 19, 1983

To prevent recurrence, corrective actions previously reported in LER 83-078 (Docket No. 50-361) concerning both a design review of the limitations of the CST automatic level control system and a proposed Technical Specification change involving the minimum level requirements for the CST, are in progress. There was no impact on the health and safety of plant personnel or the public associated with this event.

If you require any additional information, please so advise.

Sincerely,

A handwritten signature in dark ink, appearing to read "J. B. Martin". The signature is written in a cursive style with a large initial "J" and "B".

Enclosure: LER No. 83-103

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)  
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement

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