

CONTROL BLOCK										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)																													
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LICENSEE CODE										LICENSE NUMBER										LICENSE TYPE										CAT 50									
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)																																							
On December 5, 1983, at 0400, with Unit 3 in Mode 1, the Auxiliary																																							
Feedwater (AFW) flow rate channel for Steam Generator E-089 was declared																																							
inoperable due to erroneously high AFW flow rate indication when no																																							
flow existed. In accordance with LCO 3.3.3.6, Table 3.3-10, Action																																							
Statement 20, action was initiated to restore the channel to operable																																							
status within 7 days. There was no impact on the health and safety of																																							
plant personnel or the public associated with this event.																																							
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE																																							
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EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.																																							
83 104 03 L 0																																							
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB PRIME COMP. SUPPLIER COMPONENT MANUFACTURER																																							
B Z Z Z 0000 N N N 1130																																							
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)																																							
The erroneous AFW flow indication was caused by a loose terminal con-																																							
nection on a current to voltage converter. The connection was tightened																																							
and the associated channel was declared operable in accordance with Pro-																																							
cedure S03-3-3.35 on December 7, 1983, at 1646. This is considered																																							
an isolated occurrence and no further corrective action is planned.																																							
PACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION																																							
B 030 NA A Operator Observation																																							
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE																																							
Z Z NA NA																																							
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION																																							
0004 NA																																							
PERSONNEL INJURIES NUMBER DESCRIPTION																																							
0004 NA																																							
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION																																							
N NA																																							
PUBLICATION ISSUED DESCRIPTION																																							
Z NA																																							
NAME OF PREPARER J. G. HAYNES																																							
PHONE 714/492-7700																																							

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PDR ADOCK 05000362  
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NRC USE ONLY

*Southern California Edison Company*

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

December 30, 1983

J. G. HAYNES  
STATION MANAGER

RECEIVED  
NRC  
SCE  
1984 JAN -4 AM 10:16  
REGION V ISS  
TELEPHONE  
(714) 492-7700

U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region V  
1450 Maria Lane, Suite 210  
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-362  
30-Day Report  
Licensee Event Report No. 83-104  
San Onofre Nuclear Generating Station, Unit 3

Pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written report and a copy of the Licensee Event Report (LER) form for an occurrence involving Limiting Condition for Operation (LCO) 3.3.3.6 associated with the Accident Monitoring Instrumentation.

On December 5, 1983, at 0400, with Unit 3 in Mode 1 at 30% power, the Auxiliary Feedwater (AFW) flow rate channel for Steam Generator E-089 was declared inoperable when operators observed AFW flow rate indicator 3FI-4725-1 for E-089 indicating a 70 gallon per minute flow rate when no flow existed. In accordance with LCO 3.3.3.6, Table 3.3-10, Action Statement 20, action was initiated to restore the channel to operable status within seven days.

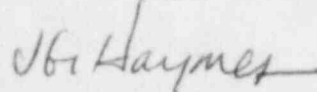
The cause of this event was a loose terminal connection on the current to voltage converter which provides the signal to 3FI-4725-1. The connection was tightened and the indicated AFW flow rate decreased to zero. The AFW flow rate channel for E-089 was declared operable in accordance with Procedure SO3-3-3.35 on December 7, 1983, at 1646. This is considered an isolated occurrence, and no further corrective action is planned. There was no impact on the health and safety of plant personnel or the public associated with this event.

11 IE 22

December 30, 1983

If you require any additional information, please so advise.

Sincerely,



J. G. HAYNES

Enclosure: LER No. 83-104

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)  
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission  
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