

CONTROL BLOCK 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 C A S O S 3 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

CONT

01 REPORT SOURCE L 6 0 5 0 0 0 3 6 2 7 1 1 1 6 8 3 8 1 2 1 9 8 3 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 On 11/16/83, at 0130, with Unit 3 in Mode 1, Containment Airborne Radi-
03 ation Monitor 3RI-7804C was found in a constant alarm state with Moni-
04 tor 3RI-7807C out of service for routine surveillance. On 11/18/83,
05 at 2107, and 12/5/83, at 1300, 3RI-7804C and 3RI-7807C were found in a
06 constant alarm state, thus rendering the Leak Detection System Gaseous
07 Channel inoperable. In accordance with LCO 3.4.5.1, grab samples of the
08 containment atmosphere every 24 hrs. were initiated. Public health and
09 safety were not affected. (See LER 83-074, Docket No. 50-362.)

09 C I 11 X 12 Z 13 I N S T R U 14 E 15 Z 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

17 LER NO. REPORT NUMBER 83
18 ACTION TAKEN E 19 FUTURE ACTION Z 20 EFFECT ON PLANT Z 21 SHUTDOWN METHOD Z 22 HOURS 000000
23 ATTACHMENT SUBMITTED N 24 NPD-4 FORM SUB. N 25 PRIME COMP. SUPPLIER A 26 COMPONENT MANUFACTURER N305
27 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

10 The constant alarm state of Monitors 3RI-7804C and 3RI-7807C was due to
11 containment radioactivity increasing to greater than the setpoints during
12 initial power ascension. New setpoints were calculated and implemented on
13 12/5/83. The Leak Detection System Gaseous Channel was operable at 2300
14 on 12/5/83, within the 30 day time constraint of LCO 3.4.5.1.

15 FACILITY STATUS B 28 0 9 0 29 NA 30 METHOD OF DISCOVERY B 31 Routine Surveillance 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 6016 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 6017 PERSONNEL EXPOSURES NUMBER 000 37 Z 38 NA 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 6018 PERSONNEL INJURIES NUMBER 000 40 NA 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 6019 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 NA 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 6020 PUBLICITY ISSUED DESCRIPTION N 44 NA 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

NAME OF PREPARER

J. G. HAYNES

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Southern California Edison Company

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SAN ONOFRE NUCLEAR GENERATING STATION 1983 DEC 22 AM 11:16

P.O. BOX 128

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REGION V

TELEPHONE
(714) 492-7700

J. G. HAYNES
STATION MANAGER

December 19, 1983

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 83-100
San Onofre Nuclear Generating Station, Unit 3

Pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written report and a copy of the Licensee Event Report (LER) form for three occurrences involving Limiting Condition for Operation (LCO) 3.4.5.1 associated with radiation monitoring instrumentation. These occurrences have been combined into a single report in accordance with NUREG-0161. This report was delayed to allow time to provide a more complete report.

On November 16, 1983, at 0130, with Unit 3 in Mode 1 at 90% power, while Containment Airborne Radiation Monitor 3RI-7807C was out of service for routine surveillance, Containment Airborne Radiation Monitor 3RI-7804C was observed to be in a constant alarm state rendering the Leak Detection System Gaseous Channel inoperable. In accordance with the action statement of LCO 3.4.5.1, actions were initiated to obtain and analyze grab samples of the containment atmosphere once per 24 hours and to return the monitor to operable status within 30 days. The alarm state was due to containment radioactivity increasing to greater than the setpoint during initial power ascension of Unit 3. The action statement was exited at 1510 on November 17, 1983, when the setpoint was raised prior to performance of a containment mini-purge. Following the purge, the setpoint was lowered.

1/1 IE-22

December 19, 1983

Similarly, on November 18, 1983, at 2107, with Unit 3 in Mode 1 at 87% power, Monitors 3RI-7804C and 3RI-7807C were observed to be in a constant alarm state. Actions pursuant to LCO 3.4.5.1 were initiated. The Action Statement was exited at 0755 on November 21, 1983.

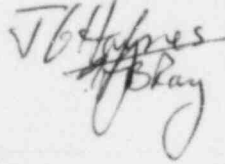
On December 5, 1983, at 1300, with Unit 3 in Mode 2, Monitors 3RI-7804C and 3RI-7807C were again observed to be in a constant alarm state. Actions pursuant to LCO 3.4.5.1 were initiated. New alarm setpoints were calculated and implemented. The monitors were returned to operable status at 2330 on December 5, 1983.

To prevent future recurrence, alarm setpoints for these monitors have been established at different activity levels to preclude simultaneous alarms which would render the Leak Detection System Gaseous Channel inoperable.

There was no impact on the health and safety of plant personnel or the public associated with this event. (See also LER 83-074, Docket No. 50-362.)

If you require any additional information, please so advise.

Sincerely,



Enclosure: LER No. 83-100

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

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Institute of Nuclear Power Operations (INPO)