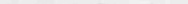


UPDATED LER - PREVIOUS REPORT DATE WAS 8-23-83  
LICENSEE EVENT REPORT

CONTROL BLOCK: 

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 MIDCIC12 000-00000000-0003 4111114

LICENSEE CODE LICENSE NUMBER LICENSE TYPE

CON'T 57 GWT 53

CON'T

REPORT SOURCE: 011 L 6 015101013115 7 0181011813 8 11211161813 9  
DOCKET NUMBER: 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80  
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES: 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

012 WHILE PERFORMING THE B AND C LEAK RATE TEST, SEVERAL CONTAINMENT ISOLATION VALVES  
013 EXHIBITED EXCESSIVE LEAKAGE CAUSING THE TOTAL LEAK RATE TO EXCEED THE LIMIT IMPOSED  
014 BY TECHNICAL SPECIFICATION 3.6.1.2.b. THE PUBLIC HEALTH AND SAFETY WERE NOT AFFECTED.  
015 PREVIOUS OCCURRENCES OF A SIMILAR NATURE INCLUDE: 50-315/82-058, 82-011, 025,  
016 79-034, 78-037, 77-011, 76-023; 316/83-016, 81-018, 79-020, 053.  
017  
018

SYSTEM CODE S 1 D 11		CAUSE CODE E 12		CAUSE SUBCODE B 13		COMPONENT CODE V I A L V I E X 14		COMP SUBCODE X 15		VALVE SUBCODE D 16	
LER/RD REPORT NUMBER 17		EVENT YEAR 8 13		SEQUENTIAL REPORT NO. 0 1 7 1 2		OCCURRENCE CODE 0 1 3		REPORT TYPE X		REVISION NO. 1	
ACTION TAKEN A 13		FUTURE ACTION Z 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 1 0 1 0 1		ATTACHMENT SUBMITTED Y 23	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 1		PRIME COMP. SUPPLIER A 25		NPRD-4 FORM SUB Y 24		COMPONENT MANUFACTURER X 9 1 9 1 9					

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

110 B AND C LEAK RATE TESTING IDENTIFIED SEVERAL VALVES AS HAVING EXCESSIVE LEAK RATES, THE  
111 LEAK RATES WERE ATTRIBUTED TO A COMBINATION OF DIRT AND SCALE DEPOSIT ON THE SEATING  
112 SURFACES, EROSION OF THE VALVE SEATS AND AN INCOMPLETE WELD AT THE ELBOW IN THE DOME  
113 WELD CHANNEL PRESSURIZATION PIPING IN ONE VOLUME. (SEE ATTACHED SUPPLEMENT)

3 9  
FACILITY STATUS (1) 5 (H) (28) % POWER (0) (0) (0) (29) OTHER STATUS (30) NA METHOD OF DISCOVERY (B) (31) SURVEILLANCE TEST DISCOVERY DESCRIPTION (32)  
8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) NA LOCATION OF RELEASE (36)  
d 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) NA  
8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
PERSONNEL INJURIES NUMBER DESCRIPTION (41) NA  
d 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43) NA  
d 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
PUBLICITY ISSUED DESCRIPTION (45) NA  
8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
NRC USE ONLY

NAME OF PREPARER R. A. PALMER

NRC USE ONLY

68 69 80

(616)465-5901

ATTACHMENT TO LER# 83-072/03X-1

SUPPLEMENT TO CAUSE DESCRIPTION

B AND C LEAK RATE TESTING IDENTIFIED SEVERAL VALVES AS HAVING EXCESSIVE LEAK RATES. THE LEAK RATES WERE ATTRIBUTED TO A COMBINATION OF DIRT AND SCALE DEPOSITS ON THE SEATING SURFACES, EROSION OF THE VALVE SEATS AND AN INCOMPLETE WELD AT THE ELBOW IN THE DOME WELD CHANNEL PRESSURIZATION PIPING IN ONE VOLUME. THE WELD CHANNEL HAD NOT PREVIOUSLY BEEN TESTED SINCE THE REQUIREMENTS OF FSAR APPENDIX Q (RESPONSE TO QUESTION 022.7) STATES THAT ONLY THE CHECK VALVE REQUIRED TO BE TESTED ON THE WELD CHANNEL PRESSURIZATION SYSTEM.

THE VALVES WERE REPAIRED BY A COMBINATION OF CLEANING AND LAPPING THE SEATING SURFACES, REPLACEMENT OF VARIOUS VALVE COMPONENTS AND BY COMPLETING THE INCOMPLETE WELD.

FINAL TEST RESULTS AFTER THE REPAIRS OF NUMEROUS VALVES RESULTED IN AN ACCEPTABLE B AND C LEAK RATE OF 0.26La WHICH IS LOWER THAN THE MAXIMUM ALLOWABLE LIMIT OF 0.6La.

D MB



**INDIANA & MICHIGAN ELECTRIC COMPANY**

DONALD C. COOK NUCLEAR PLANT  
P.O. Box 458, Bridgman, Michigan 49106  
(616) 465-5901

December 16, 1983

Mr. J.G. Keppler, Regional Administrator  
United States Nuclear Regulatory Commission  
Region III  
799 Roosevelt Road  
Glen Ellyn, IL 60137

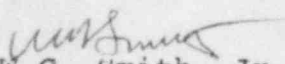
Operating License DPR-58  
Docket No. 50-315

Dear Mr. Keppler:

Pursuant to the requirements of the Appendix A Technical  
Specifications, the following report/s are submitted:

RO 83-072/03X-1

Sincerely,

  
W.G. Smith, Jr.  
Plant Manager

/cbm

Attachment

cc: John E. Dolan  
M.P. Alexich  
R.F. Kroeger  
H. Brugger  
E.R. Swanson, RO:III  
R.C. Callen, MPSC  
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J.M. Hennigan  
R.O. Bruggee, EPRI  
INPO  
PNSRC  
J.F. Stietzel  
E.L. Townley  
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DEC 19 1983

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