NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | Licensee Code: N C B E P 1 (2) | License Number: 0 0 - 0 0 0 0 0 - 0 0 (3) | License Type: 4 1 1 1 1 (4) | CAT (5)

CON'T 01 | Report Source: L (6) | Docket Number: 0 5 0 - 0 3 2 5 (7) | Event Date: 1 1 2 7 8 3 (8) | Report Date: 1 2 2 7 8 3 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During a unit reactor startup, reactor power intermediate range monitor (IRM), C51-IRM-K601H was deter-
03 mined to be inoperable due to downscale indications. At the time, IRM K601A had been inoperable since Aug-
04 ust 10, 1983, due to intermittent indications and IRM K601D had been inoperable since November 16, 1983,
05 as a control rod block input due to incomplete surveillance requirements. This event rendered the IRM
06 control rod block function inoperable due to less than six (6) operable IRMs, the specified minimum re-
07 quired number of operable IRMs. This event did not affect the health and safety of the public.
08 Technical Specifications 3.3.4, 6.9.1.9b

09 | System Code: I B (11) | Cause Code: E (12) | Cause Subcode: E (13) | Component Code: I N S T R U (14) | Comp. Subcode: Y (15) | Valve Subcode: Z (16)

(17) LER/RO Report Number: Event Year 8 3 | — | Sequential Report No. 0 6 1 | / | Occurrence Code 0 3 | Report Type L | — | Revision No. 0

Action Taken: X (18) | Future Action: Z (19) | Effect on Plant: C (20) | Shutdown Method: Z (21) | Hours: 0 0 0 0 (22) | Attachment Submitted: N (23) | NPRD-4 Form Sub.: Y (24) | Prime Comp. Supplier: N (25) | Component Manufacturer: G 0 8 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The signal preamplifier low/high range gain of K601H, Part No. 194X672G8, was out of adjustment. The
11 problem with K601A is attributed to a defective instrument cable. K601D was inoperable as a control rod
12 block input because the monitor detector position control rod block functional test, PT-01.10, was pending
13 performance. PT-01.10 was satisfactorily completed and the system was returned to service within 2 hours
14 and 36 minutes of the event.

15 | Facility Status: C (28) | % Power: 0 0 3 (29) | Other Status (30): NA | Method of Discovery: A (31) | Discovery Description (32): Operational Event

16 | Activity Released: Z (33) | Content of Release: Z (34) | Amount of Activity (35): NA | Location of Release (36): NA

17 | Personnel Exposures — Number: 0 0 0 (37) | Type: Z (38) | Description (39): NA

18 | Personnel Injuries — Number: 0 0 0 (40) | Description (41): NA

19 | Loss of or Damage to Facility (43) — Type: Z (42) | Description: NA

20 | Publicity — Issued: N (44) | Description (45): NA | NRC USE ONLY

8401040315 831227
PDR ADOCK 05000325
S PDR

NAME OF PREPARER M. J. Pastva, Jr. PHONE: 919-457-9521

GPO 917-926

CP&L

Carolina Power & Light Company

83 DEC 29 A8:58

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

December 27, 1983

FILE: B09-13510C
SERIAL: BSEP/83-4037

Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-83-61

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/joh/LETJH1

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

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