

CONTROL BLOCK:								1
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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	C	A	S	0	S	3	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	4			5
9	14						15	25										26	30			37	58				
		LICENSEE CODE						LICENSE NUMBER										LICENSE TYPE			CAT						

CON'T

REPORT SOURCE L 6 0 5 0 0 0 3 6 2 7 1 1 1 6 8 3 8 1 2 1 5 8 3 9

60 61 DOCKET NUMBER 66 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

On 11/16/83, with Unit 3 in Mode 1 at 90% power, channel A of the post Loss of Coolant Accident (LOCA) hydrogen monitor failed and was declared inoperable. In accordance with the Action Statement of LCO 3.6.4.1, action was initiated to restore the monitor to operable status within 30 days. There was no impact on the health and safety of plant personnel or the public.

0	8		
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SYSTEM CODE		CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE						COMP. SUBCODE	VALVE SUBCODE								
0	9	S	E	11	E	12	G	13	I	N	S	T	R	U	14	P	15	Z	16
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26

(17)	LER / RO REPORT NUMBER	[ 8   3 ] 21      22	[ — ] 23	SEQUENTIAL REPORT NO.	[ 1   0   2 ] 24      25      26	[ / ] 27	SUCCESS CODE	[ 0   3 ] 28      29	TYPE	[ L ] 30	[ — ] 31	NO.	[ 0 ] 32
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER					
[ A ] (18)	[ Z ] (19)	[ Z ] (20)	[ Z ] (21)	[ 0   0   0   0 ] 33      34      35      36	[ N ] (23)	[ N ] (24)	[ A ] (25)	[ G   0   8   0 ] 37      38      39      40					

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)	
1 0	Inoperability of the monitor was due to a shorted capacitor in a circuit
1 1	board of the power supply causing a blown fuse. The circuit board and
1 2	fuse were replaced and the monitor was declared operable in accordance
1 3	with procedure S023-II-1.11 on 11/17/83. This is considered an isolated
1 4	occurrence and no further corrective action is planned.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION					
1	5	B	28	0	9	0	29	NA	30	A	31	Operator Observation	32

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

ACTIVITY CONTENT  
RELEASED OF RELEASE

AMOUNT OF ACTIVITY (35)

LOCATION OF RELEASE (36)

1 6 Z (33) Z (34) NA NA

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION
1	7	0	0	NA

PERSONNEL INJURIES  
NUMBER DESCRIPTION (41)

1 8 0 0 0 (40) NA

8401040000

1 9 [Z] (42) NA

PUBLICITY  
ISSUED DESCRIPTION (45) NA

NRC USE ONLY

NAME OF PREPARER

J. G. HAYNES

PHONE: 714/492-7700

IE 22

RECEIVED  
NRC

*Southern California Edison Company*

SAN ONOFRE NUCLEAR GENERATING STATION  
P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

**SCE**

J. G. HAYNES  
STATION MANAGER

December 15, 1983

TELEPHONE  
(714) 492-7700

U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region V  
1450 Maria Lane, Suite 210  
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-362  
30-Day Report  
Licensee Event Report No. 83-102  
San Onofre Nuclear Generating Station, Unit 3

Pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written report and a copy of the Licensee Event Report (LER) form for an occurrence involving Limiting Condition for Operation (LCO) 3.6.4.1 associated with hydrogen monitors.

On November 16, 1983, Channel A of the Post Loss of Coolant Accident (LOCA) hydrogen monitor failed and was declared inoperable. In accordance with the Action Statement of LCO 3.6.4.1, action was initiated to restore the monitor to operable status within 30 days. The other hydrogen monitor remained operable.

Investigation of the failure revealed a shorted capacitor on a circuit board in the power supply which caused a blown fuse. The circuit board and fuse were replaced and the monitor was declared operable on November 17, 1983, in accordance with Surveillance Procedure SO23-II-1.11.

There was no impact on the health and safety of plant personnel or the public during this event.

1/1 7022

Mr. J. B. Martin

-2-

December 15, 1983

If you require any additional information, please so advise.

Sincerely,

*JG Haynes*

Enclosure: LER No. 83-102

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)  
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement

U. S. Nuclear Regulatory Commission  
Division of Technical Information and Document Control

Institute of Nuclear Power Operations (INPO)