

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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REPORT SOURCE

L	6	0	5	0	-	0	3	2	5	7	1	2	0	3	8	3	8	1	2	3	0	8	3	9
60	61	DOCKET NUMBER					68	69	EVENT DATE					74	75	REPORT DATE					80			

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | Routine surveillance during unit power operation revealed RWCS differential flow indica-

0 3 | tor, 1-G31-R615, was showing an erroneous indication of RWCS differential flow. During

0 4 | subsequent unit power operation on December 7, 1983, spurious RWCS "Leak Hi-Hi" alarm

0 5 | annunciations occurred. Neither event affected the health and safety of the public.

0 6 |

0 7 |

Technical Specifications Table 3.3.2-1, Item 3a, 6.9.1.9b

SYSTEM CODE S D 11		CAUSE CODE D 12		CAUSE SUBCODE Z 13		COMPONENT CODE I N S T R U 14		COMP. SUBCODE T 15		VALVE SUBCODE Z 16							
EVENT YEAR 8 3		SEQUENTIAL REPORT NO. 0 6 3		OCCURRENCE CODE 0 3		REPORT TYPE L		REVISION NO. 0									
ACTION TAKEN X 18		FUTURE ACTION G 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 0 22		ATTACHMENT SUBMITTED Y 23		NPRD-4 FORM SUB. Y 24		PRIME COMP. SUPPLIER N 25		COMPONENT MANUFACTURER G 0 8 0 26	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The events occurred due to entrapped air in the sensing lines of the RWCS leak detection

1 1 system. This resulted from a procedural inadequacy in the RWCS high flow response time

1 2 test, PT-45.2.16, which was performed respectively on December 3 and 6, 1983. The en-

1 3 trapped air was removed and the RWCS leak detection system was returned to service.

1 4 Appropriate revisions to PT-45.2.16 will be implemented by January 15, 1984, to help pre-

7 8 9 vent future similar occurrences. (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

FACILITY STATUS % POWER OTHER STATUS

1 5 E (28) 1 0 0 (29) NA A (31) Operator Surveillance

ACTIVITY CONTENT
RELEASED OF RELEASE

AMOUNT OF ACTIVITY (35)
NA

LOCATION OF RELEASE (36)

PERSONNEL EXPOSURES

NUMBER			TYPE	DESCRIPTION
1	7	0 0 0	(37) Z	(38) NA

PERSONNEL INJURIES					
NUMBER			DESCRIPTION		
1	8		0	0	0
			(40)	NA	

1 9 11 12
LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
Z (42)
1 9 3 13
8401060361 831230 NA
IE22

PUBLICITY
 ISSUED DESCRIPTION (45)
 2 0 N 44 NA
 PDR ADOCK 05000325
 S PDR
 NRC USE ONLY

NAME OF PREPARER M. J. Pastva, Jr.

PHONE: 919-457-9521

LER ATTACHMENT - RO # 1-83-63

Facility: BSEP Unit No. 1

Event Date: December 3, 1983

Routine Control Room backpanel checks during unit power operation revealed that Reactor Water Cleanup System (RWCS) differential flow indicator 1-G31-R615 was showing an indication of 40 gpm. A comparative check of RWCS inlet and outlet flows determined that the indication was erroneous. On December 7, 1983, a spurious, RWCS "Leak Hi-Hi" alarm annunciation occurred. Neither of these events affected the health and safety of the public.

These events resulted from entrapped air in the RWCS leak detection instrumentation. Air was introduced into the instrument sensing lines of the affected RWCS instruments, 1-G31-FT-N012, N036, and N041, during respective performances of the RWCS high flow response time test, PT-45.2.16, on December 3 and 6, 1983. This resulted from a deficiency in the PT procedure where sensing line drains to FT-N012 and N036 are opened in accordance with the procedure.

FT-N012, N036, and N041, one on the RWCS suction line and one each on the two RWCS discharge lines, feed an RWCS summing circuit, 2-G31-FY-K604. K604 takes the RWCS suction flow and compares it with the combined RWCS discharge flow to produce a differential flow signal to the RWCS isolation circuits. 2-G31-R615 is a direct reading indicator off one of the two parallel input signals from FY-K604.

In each case, the entrapped air was removed from the affected RWCS instruments and the RWCS leak detection system was returned to service.

As a result of these events, by January 15, 1984, appropriate revisions to PT-45.16.2 will be made and implemented in order to help prevent future similar occurrences.



Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
December 30, 1983

FILE: B09-13510C
SERIAL: BSEP/83-4051

Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-83-63

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/ag/LETJ03

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

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