NRC FORM 366
(7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	A L B R F 3 (2)	0 0 - 0 0 0 0 0 - 0 0 (3)	4 1 1 1 1 (4)	(5)		
	LICENSEE CODE	LICENSE NUMBER	LICENSE TYPE	CAT		

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0 1	REPORT SOURCE L (6)	0 5 0 0 0 2 9 6 (7)	1 2 1 4 8 3 (8)	1 2 2 7 8 3 (9)
		DOCKET NUMBER	EVENT DATE	REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During the unit 3 cycle 5 refueling outage, 9 of 13 2-stage

0 3 relief valves tested for as-found set pressure initially failed to

0 4 lift within the allowable 1% tolerance (T.S. 2.2.A). The total

0 5 average deviation is 3.72%. There was no effect on the health or

0 6 safety of the public. Per previous calculations performed by the

0 7 General Electric Company, a 5% deviation would have no significant

0 8 effect on nuclear safety.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	C C (11)	E (12)	X (13)	V A L V E X (14)	X (15)	B (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 6 0	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	X (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	L (25)	T 0 2 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 As stated in previous similar LERs, the cause is being evaluated by

1 1 the manufacturer. When the cause is determined by the manufacturer,

1 2 a followup report will be issued. The nine Target Rock Model

1 3 7567F-100 safety relief valves that failed will be reset and

1 4 retested prior to installation.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	H (28)	0 0 0 (29)	NA	B (31)	Routine testing

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	NA

8401040237 831227
PDR ADOCK 05000296
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LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	NA	

NAME OF PREPARER E. T. Holder PHONE (205) 729-0885

Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/18/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 83060 Technical Specification Involved 2.2.A

Reported Under Technical Specification 6.7.2.a.1 * Date Due NRC 12/28/83

Event Narrative:

Units 1 and 3 were in refueling outages and unit 2 was operating normally at 95% power. Unit 3 was the only unit affected by this event. While performing bench tests at Wyle Laboratories on the main steam relief valves per Technical Specification 4.6.D.1, nine of the 13 valves' as-found set pressure initially failed to lift within the allowable 1% or ± 11 psig tolerance (T.S. 2.2.A). These valves will be reset and retested prior to installation. The cause is being evaluated with the manufacturers for the Target Rock Model 7567F-100 valves. When the cause is determined by the manufacturer, a followup report will be submitted. The total average deviation for these nine valves is 3.75%. Previous calculations by General Electric Company indicated that a total average deviation in relief valve setpoint of 5.0% could have existed with no significant effect on nuclear safety. The evaluations performed in LER BFRO-50-296/81074 remain valid in that this problem does not result in an overpressurized condition of reactor vessel piping nor does it result in any appreciable increase in MCPR operating limit. There was no effect on the public health and safety. Tabulated on the attachment are the valve serial numbers, set pressures, as-found set pressures, and percent deviation for the first lift of these nine MSRVs.

* Previous Similar Events:

BFRO 50-260/8227
296/8174
259/8125
296/8054
260/8040
259/8336

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

ATTACHMENT TO LER 296/83060

Serial Number	Set Pressure	As-Found Set Pressure	Percentage Deviation
1018	1115 ± 11 psig	1130 psig	1.3
1019	1125 ± 11 psig	1161 psig	3.2
1023	1105 ± 11 psig	1121 psig	1.4
1024	1125 ± 11 psig	1179 psig	4.8
1029	1125 ± 11 psig	1176 psig	4.5
1030	1115 ± 11 psig	1158 psig	3.9
1063	1105 ± 11 psig	1197 psig	8.3
1078	1125 ± 11 psig	1149 psig	2.1
1085	1125 ± 11 psig	1170 psig	4.0

The total average deviation for these nine valves was 3.72%.

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

83 DEC 29 A8:58

December 27, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW.
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE REPORT BFRO-50-296/83060

The enclosed report provides details concerning failure of nine of thirteen 2-stage relief valves to lift within the allowable 1-percent tolerance when tested for as-found test pressure. This report is submitted in accordance with Browns Ferry unit 3 Technical Specification 6.7.2.a(1).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

for H. J. Green
Director of Nuclear Power

Enclosure
cc (Enclosure):
Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
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NRC Inspector, Browns Ferry

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