

LICENSEE EVENT REPORT

CONTROL BLOCK:										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)									
<div> <div>01</div> <div>M I P A L 1</div> <div>200-000000-00</div> <div>3411111</div> <div>4</div> <div>5</div> </div>										<div> <div>7</div> <div>8</div> <div>9</div> <div>14</div> <div>15</div> <div>25</div> <div>26</div> <div>30</div> <div>57</div> <div>58</div> </div>									
<div> <div>CON'T</div> <div>01</div> </div>										<div> <div>REPORT SOURCE</div> <div>605000255</div> <div>61</div> <div>66</div> <div>69</div> <div>EVENT DATE</div> <div>74</div> <div>75</div> <div>REPORT DATE</div> <div>80</div> </div>									
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)																			
<div> <div>03</div> <div>During the routine shutdown evolution to begin the current refueling outage,</div> </div>																			
<div> <div>03</div> <div>the requirements of hot standby condition per TS 1.1 and 3.1.3.c were viola-</div> </div>																			
<div> <div>04</div> <div>ted resulting in the reactor being in an undefined condition. With control</div> </div>																			
<div> <div>05</div> <div>rods withdrawn and the reactor at less than 1% power, but still critical per</div> </div>																			
<div> <div>06</div> <div>TS definition (10⁻⁴%), Tavg fell below 525° to approx 512°. As a result,</div> </div>																			
<div> <div>07</div> <div>neither the requirements for hot standby, hot shutdown nor cold shutdown were</div> </div>																			
<div> <div>08</div> <div>satisfied. Reportability per TS 6.9.2.b(3) determined on November 23, 1983.</div> </div>																			
<div> <div>09</div> <div> <div>SYSTEM CODE</div> <div>9</div> <div>10</div> <div>CAUSE CODE</div> <div>11</div> <div>12</div> <div>CAUSE SURCODE</div> <div>13</div> <div>14</div> <div>COMP SUBCODE</div> <div>15</div> <div>16</div> <div>VALVE SUBCODE</div> <div>17</div> <div>18</div> <div>19</div> <div>20</div> <div>21</div> <div>22</div> <div>23</div> <div>24</div> <div>25</div> <div>26</div> <div>27</div> <div>28</div> <div>29</div> <div>30</div> <div>31</div> <div>32</div> <div>33</div> <div>34</div> <div>35</div> <div>36</div> <div>37</div> <div>38</div> <div>39</div> <div>40</div> <div>41</div> <div>42</div> <div>43</div> <div>44</div> <div>45</div> <div>46</div> <div>47</div> </div> </div>																			
<div> <div>17</div> <div>LER RO REPORT NUMBER</div> <div>21</div> <div>22</div> <div>EVENT YEAR</div> <div>23</div> <div>24</div> <div>25</div> <div>26</div> <div>27</div> <div>28</div> <div>29</div> <div>30</div> <div>31</div> <div>32</div> <div>33</div> <div>34</div> <div>35</div> <div>36</div> <div>37</div> <div>38</div> <div>39</div> <div>40</div> <div>41</div> <div>42</div> <div>43</div> <div>44</div> <div>45</div> <div>46</div> <div>47</div> </div>																			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)																			
10 Actions initiated to compensate for problems experienced with the turbine																			
11 bypass valve resulted in a Tavg decrease. The reactor is believed to have																			
12 been in a safe condition; evaluation continues. A TS change will be inves-																			
13 tigated to define reactor conditions between currently defined conditions.																			
14 Until then, operators will be instructed to adhere to TS definitions.																			
15 FACILITY STATUS 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47																			
16 ACTIVITY RELEASED 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47																			
17 PERSONNEL EXPOSURES 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47																			
18 PERSONNEL INJURIES 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47																			
19 LOSS OF OR DAMAGE TO FACILITY 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47																			
20 PUBLICITY ISSUED 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47																			
21 NRC USE ONLY 48 49 50 51 52 53 54 55 56 57 58																			

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**Consumers
Power
Company**

David J VandeWalle
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Nuclear Licensing*

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December 23, 1983

James G Keppler, Administrator
Region III
US Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 83-077 -
UNDEFINED OPERATING REGION

On the reverse side is Licensee Event Report (LER) 83-077 (Undefined Operating Region). Reportability is based on Technical Specification 6.9.2.b(3).

David J VandeWalle
Director, Nuclear Licensing

CC Director, Office of Nuclear Reactor Regulation
Director, Office of Inspection and Enforcement
NRC Resident Inspector-Palisades

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DEC 27 1983

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