

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

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O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

January 6, 1983

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
Feedwater Isolation Valve
Technical Specification
Justification

Dear Mr. Denton:

On November 16, 1983, South Carolina Electric and Gas Company (SCE&G) requested an amendment to the Virgil C. Summer Nuclear Station Technical Specifications which addressed the Feedwater Isolation Valve(s). The Staff, in a subsequent conversation with SCE&G, requested additional information for justification of the seventy-two (72) hour feedwater isolation valve inoperable time limit.

As shown in the Final Safety Analysis Report (FSAR) Figure 7.2-1, Sheet 13, the Virgil C. Summer Nuclear Station Reactor Protection System (RPS) logic dictates that on receipt of a Safety Injection (SI) or Steam Generator Hi-Hi Level signal, the following feedwater related actions occur:

- 1) Feedwater Isolation Valve Closure
- 2) Feedwater Control and Bypass Valve Closure
- 3) Feedwater Pump Trip and its associated discharge valve closure

The steamline break events described in FSAR Sections 6.2.1.3.10.3.1, 6.2.1.3.10.3.3 and 15.4.2 take credit for feedwater isolation. Feedwater isolation, as assumed in these analyses, can be accomplished by any one of the above three (3) feedwater related actions. Therefore, a feedwater isolation valve inoperable time limit of 72 hours is acceptable since Feedwater Pump Trip and Feedwater Control Valve Closure provide added protection on receipt of an SI or Steam Generator Hi-Hi Level signal.

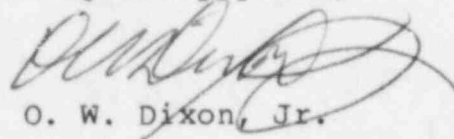
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If there should be any questions, please contact us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

WRM:OWD/fjc

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