

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'TEVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	5
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0	7
---	---

0	8	
7	8	

0 9 8

SYSTEM CODE: H I 11

CAUSE CODE: E 12

CAUSE SUBCODE: A 13

COMPONENT CODE: V A L V O P 14

COMP. SUBCODE: D 15

VALVE SUBCODE: Z 16

(17) LER/RO REPORT NUMBER 8 3 —

SEQUENTIAL REPORT NO. 1 7 2 /

OCCURRENCE CODE 0 3

REPORT TYPE L —

REVISION NO. 0

ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS				ATTACHMENT SUBMITTED		NPD-4 FORM 505		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER		
E	18	Z	19	Z	20	Z	21	0	0	0	0	N	23	N	24	N	25	M	1	2

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

Investigation revealed the value limit switch set at minimum

Investigation revealed the valve limit switch out of adjustment. The limit switch

was adjusted and the valve returned to service at 0900 (C) on 11/30/83 within the

4-hour time limit of the action statement. No further corrective action is required.

13

69
FALITY

STATUS	% POWER	OTHER STATUS	REASON FOR DISCOVERY	DISCOVERY DESCRIPTION

1	5	E	(24)	0	9	9	(79)	NA	A	(31)	Operator observation
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8	9	10	12	17	44	45	46
ACTIVITY	CONTENT						

IDENTITY	CONTENT	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
RELEASED	OF RELEASE	(35)	(36)

1	G	Z	(33)	Z	(34)	NA	NA
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8 9 10 11 44 45

PERSONNEL EXPOSURES		
NUMBER	TYPE	DESCRIPTION (30)

1	7	0	0	0	(37)	Z	(38)	NA
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8 9 11 12 13

PERSONNEL INJURIES	8401050851 851228
NUMBER DESCRIPTION (41)	PDR ADDCK 05000328

1	8	0	0	0	(40)	NA	S	PDR	✓
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8 9 11 12

[illegible]

9	2	(42)	NA
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8 9 10

ISSUED: DESCRIPTION (45) PUBLICITY NRC: USE ONLY

[illegible]

10 68 40

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

December 28, 1983 P2:31

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW.
Atlanta, Georgia 30303

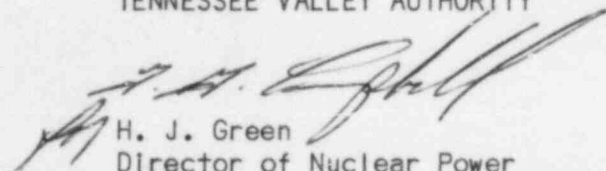
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 2 - DOCKET
NO. 50-328 - FACILITY OPERATING LICENSE DPR-79 - REPORTABLE OCCURRENCE
REPORT SQRO-50-328/83172

The enclosed report provides details concerning the inoperability of steam
generator blowdown containment isolation valve 2-FCV-1-182. This report
is submitted in accordance with Sequoyah unit 2 Technical Specification
6.9.1.13.b.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

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Washington, D.C. 20555

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NRC Inspector, Sequoyah

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