

APPROVED BY OMB
3150-0011
EXPIRES 4-30-82

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 SCVCS 1 00 - 00 000 - 00 3 4 100 0 5

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

LICENSEE NAME CODE LICENSE NUMBER LICENSE TYPE CAR

CON'T

0 1

REPORT SOURCE L 6 0 5 0 0 0 3 9 5 7 1 2 0 1 8 3 8 1 2 2 7 8 3 9

60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

During a pipe support inspection of the Residual Heat Removal System, December 1,

1983, with the Plant in Mode 5, supports RHH-134, 135, and 137 were declared

inoperable. RHH-134 had a bent paddle plate and the other two supports (135 and

137) pipe clamps were out of tolerance. There were no adverse consequences due

to this event. The Licensee complied with Action Statement (a) of Technical

Specification 3.7.7, "Snubbers."

0	8																			80
7	8	SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE						
C F		11	E		12	B		13	S U P P O R T				14	D		15	Z		16	
9	10	11	12	13	14	15	16	17	18	19	20									
0	9	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.										
7	8	8 3		1 3 5		0 3		L		0										
9	10	11	12	13	14	15	16	17	18	19	20									
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER				
A		18	X		19	Z		20	0 0 0		N		23	N		24	A		25	
21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	

The system's remaining supports and components were reviewed by Nuclear

Engineering and found acceptable. The supports were functionally tested prior to

repair and only RHH-137 failed. The supports were repaired, tested, and declared

operable on December 2, 1983. Engineering continues to evaluate the cause of

| this event.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
1	5	G	28	0	0	0	29	N/A	30
ACTIVITY CONTENT RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE					
1	6	Z	33	Z	34	N/A	35	N/A	36

PERSONNEL EXPOSURE		DESCRIPTION	
NUMBER	TYPE		
1 7	0 0 0	37	38
			N/A

PERSONNEL INJURIES		DESCRIPTION	
NUMBER			
1	8	000	(40) N/A
		8401050597 831227	

1	9	Z	42	N/A	PDR ADOCK 05000395 S PDR
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PUBLICATION		ISSUED		DESCRIPTION		NRC USE ONLY	
2	0	N	44	N/A			

NAME OF PREPARER

PHONE

(803) 345-5209

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

31 JAN 3 1983
December 27, 1983

Mr. James P. O'Reilly
Regional Administrator
U.S. Nuclear Regulatory Commission
Region II, Suite 2900
101 Marietta Street, N.W.
Atlanta, Georgia 30303

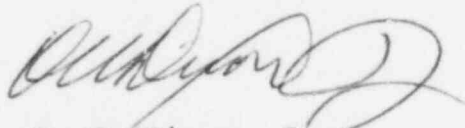
SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
Thirty Day Written Report
LER 83-135

Dear Mr. O'Reilly:

Please find attached Licensee Event Report #83-135 for the Virgil C. Summer Nuclear Station. This Thirty Day Report is required by Technical Specification 6.9.1.13.(b) as a result of entry into Action Statement (a) of Technical Specification 3.7.7, "Snubbers," on December 1, 1983.

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

RJB:OWD/mac
Attachment

cc: V. C. Summer
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