

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'TREPORT  
SOURCE

60	61	DOCKET NUMBER					68	EVENT DATE					74	REPORT DATE					80					
L	6	0	5	0	-	0	3	2	5	7	1	1	2	7	8	3	8	1	2	2	2	8	3	9

0 2 | While withdrawing control rods during a unit reactor startup and subsequent unit load changes during

0 3 | power operation, the following control rods with control board position indication problems were

0 4 | identified: rod 14-19 (no "tens" digit position indicated at "01", "11", "21", "31", "41"), rod 22-03

0 5 | (no indication at "05"), rod 38-31 (no indication at "28"). In each case the health and safety of

0 6 | the public were not affected.

0 7 |

08 | Technical Specifications 3.1.3.7, 6.9.1.9b 80

09		SYSTEM CODE I B		11	CAUSE CODE X		12	CAUSE SUBCODE Z		13	COMPONENT CODE X X X X X X						14	COMP. SUBCODE Z		15	VALVE SUBCODE Z		16
7	8	9	10		11		12		13						14		15		16				
17		LER/RO REPORT NUMBER		EVENT YEAR 8 3		21	22	SEQUENTIAL REPORT NO. 0 5 6		24	25	26	OCCURRENCE CODE /		27	REPORT TYPE 1		30	REVISION NO. 0		32		
ACTION TAKEN X		FUTURE ACTION X		EFFECT ON PLANT Z		SHUTDOWN METHOD Z		HOURS 0 0 0 0		ATTACHMENT SUBMITTED N		NPRD-4 FORM SUB. Y		PRIME COMP. SUPPLIER N		COMPONENT MANUFACTURER G 0 8 0							
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53			

1 0 Initial troubleshooting shows that defective position indication probe (PIP) connectors or read switches

1 1 in the subject rods most likely caused the observed indication problems. These indication problems

1 2 will be further investigated and resolved during a future unit outage.

1 3

1 4

8 9 FACILITY STATUS 30 METHOD OF DISCOVERY DISCOVERY DESCRIPTION 32

1 5 C 28 0 0 0 29 NA A 31 Operator Surveillance

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

ACTIVITY CONTENT  
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 Z (33) NA 44 45

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	Z	(38)		NA

8401030327 831222  
PDR ADOCK 05000325  
S PDR

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	8	0	0	0	40
				NA	

LOSS OF OR DAMAGE TO FACILITY (43)  
TYPE DESCRIPTION NA  
1 9 Z (42) 10

7 8 9 10 68 69 8

PUBLICATION ISSUED DESCRIPTION NA NRC USE ONLY

2 0 N 44 45

PHONE: 919-457-9521

NRC USE ONLY

0007-1226(200809)28:3;1-B

CP&L

Carolina Power & Light Company

Brunswick Steam Electric Plant  
P. O. Box 10429  
Southport, NC 28461-0429  
December 22, 1983

FILE: B09-13510C  
SERIAL: BSEP/83-3996

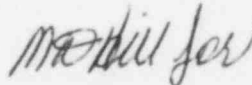
Mr. James P. O'Reilly, Administrator  
U. S. Nuclear Regulatory Commission  
Region II, Suite 3100  
101 Marietta Street N.W.  
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1  
DOCKET NO. 50-325  
LICENSE NO. DPR-71  
LICENSEE EVENT REPORT 1-83-56

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,



C. R. Dietz, General Manager  
Brunswick Steam Electric Plant

RMP/pms/LETPS1

Enclosure

cc: Mr. R. C. DeYoung  
NRC Document Control Desk

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