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NRC USE ONLY

IE22

Tennessee Valley Authority  
Browns Ferry Nuclear Plant

Form BF 17  
BF 15.2  
2/17/82

## LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 83005 Technical Specification Involved 4.7.A.2.b  
Reported Under Technical Specification 6.7.2.b.(4) \* Date Due NRC 12/23/83

Event Narrative:

On February 16, 1983, unit 1 was operating at 95-percent power, unit 3 was operating at 100-percent power, and unit 2 was in a refueling outage. During the integrated leak rate test on unit 2, the primary containment leak rate was found to exceed the allowable leak rate of 0.75 La (1.5 percent/Day - Technical Specification 4.7.A.2.b). This leakage was below the design limit of 2 percent La. A leak was found on 2-FCV-64-20 at the flange gasket and flange bolts were tightened to stop the leak. On February 19, 1983, the primary containment leak rate was reduced to approximately .5 percent/Day. There was no danger to the health and safety of the public. No redundant systems were required to be in service and operable.

A modification to the flange on this valve and similar valves will be made to allow testing the flange gaskets for leakage. This modification was made to valves in unit 1. The valves in units 2 and 3 are scheduled to be modified during the cycle 5 refueling outages.

\* Previous Similar Events:

None

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

\*Revision: JRR

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

DEC 28 1983 8:45

December 22, 1983

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW.  
Atlanta, Georgia 30303

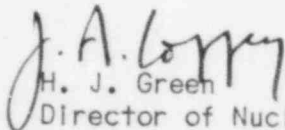
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET  
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE  
REPORT BFR0-50-260/8305 - REVISION 1

The enclosed report is a supplement to my letter to you dated March 16,  
1983 concerning primary containment leakage in excess of the allowed rate  
discovered during an integrated leak rate test. This report is submitted  
in accordance with Browns Ferry unit 2 Technical Specification 6.7.2.b(4).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
H. J. Green  
Director of Nuclear Power

Enclosure

cc (Enclosure):

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U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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Institute of Nuclear Power Operations  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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