

## LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | T | N | S | N | P | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | | | 5

3 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 32

CON'T

REPORT  
SOURCE

REPORT SOURCE L 6 0 5 0 0 0 3 2 7 7 1 1 2 4 8 3 8 1 2 2 2 8 3 9  
60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 With unit i in mode 3 (0% Rx power) at 1925 CST at 11/24/83, one steam generator

03 flow channel 1-FT-1-28A was declared inoperable due to reading high. This event

0 4 required entry into action statement (7) of LCO 3.3.1.1. There was no effect

05 | upon public health and safety. Previous occurrences - none.

0 5

07

0	8											
7	8	9										

0 1 9

I A 11

E 12

E 13

I N S T R U 14

T 15

Z 16

17 LER/RO REPORT NUMBER [ 8 | 3 ] [ 21 | 22 ] [ — ] [ 23 ]

SEQUENTIAL REPORT NO. [ 1 | 7 | 6 ] [ 24 | 25 | 26 ] [ / ] [ 27 ]

OCCURRENCE CODE [ 0 | 3 ] [ 28 | 29 ]

REPORT TYPE [ L ] [ 30 ] [ — ] [ 31 ]

REVISION NO. [ 0 ] [ 32 ]

ACTION TAKEN 18 FUTURE ACTION X 9 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0000 22 ATTACHMENT SUBMITTED N 23 NPD-4 FORM 508 N 24 PRIME COMP. SUPPLIER L 25 COMPONENT MANUFACTURER B 0 8

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Troubleshooting the inoperable channel revealed the delta-pressure (d/p) unit to

11 be ruptured. The cause is unknown and the transmitter has been returned to Barton

1 2 for review and repair. Presently, the unit is in an outage and the transmitter

1 3 will be replaced and operable before startup.

14

FACILITY STATUS		% POWER			OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION					
1	5	G	(78)	0	0	0	(29)	NA	B	(23)	Operator observation	(72)

8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80	81	82	83	84	85	86	87	88	89	90	91	92	93	94	95	96	97	98	99	100
ACTIVITY CONTENT										RELEASED OF RELEASE										AMOUNT OF ACTIVITY										LOCATION OF RELEASE																																																														

1 G 2 Z 3 33 4 Z 5 34 6 NA 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 NA 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

PERSONNEL EXPOSURES									
NUMBER			TYPE		DESCRIPTION				
1	7	0	0	0	(37)	7	(38)	NA	

8	9	11	12	13	8
PERSONNEL INJURIES					
NUMBER	DESCRIPTION				
	(1)				

NO.		DESCRIPTION	
1	8	0 0 0	NA

LOSS OF OR DAMAGE TO FACILITY		DESCRIPTION		PDR	
TYPE	NA	DESCRIPTION	PDR	ADOCK	ADOCK
9	2	NA	S	8401030071	831222
			PDR	ADOCK	05000327

8 9 10  
PUBLICITY  
ISSUED: RESEARCH (AS)  
NBC USE ONLY

ISSUED DESCRIPTION NA

Name of Preparer: H. R. Rogers /M. R. Harding

Phone: (615) 870-6422

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

DEC 28 8:44

December 22, 1983

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW.  
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET  
NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE  
REPORT SQRO-50-327/83176

The enclosed report provides details concerning the inoperability of one  
reactor protection steam flow channel. This report is submitted in  
accordance with Sequoyah unit 1 Technical Specification 6.9.1.13.b.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

*J. A. Coffey*  
H. J. Green  
Director of Nuclear Power

Enclosure

cc (Enclosure):

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Washington, D.C. 20555

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NRC Inspector, Sequoyah

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