

APPROVED BY OMB  
3150-0011  
EXPIRES 4-30-82

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

09		SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22
		I	B	E		E		I	N	S	T	R	U	T	Z
17		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.					
LER/RO REPORT NUMBER		21	22	23	24	25	26	27	28	29	30	31	32		
		8	3	—	0	6	5	/	0	3	L	—	0		
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER							
18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33
E	X	Z	Z	0	0	0	0	Y	Y	Y	L	B	0	8	0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

7 8 9  
FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

1 5 C (28) 0 0 0 (29) N/A C (31) Special Investigation

10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 Z (33) Z (34) N/A N/A

PERSONNEL EXPOSURES		TYPE		DESCRIPTION
NUMBER				
1	7	0	0	0
		37	Z	38
				N/A

PERSONNEL INJURIES		DESCRIPTION	
NUMBER			
1	8	0	0
0	0	0	40
N/A			

8 9 11 12  
LOSS OF OR DAMAGE TO FACILITY  
TYPE DESCRIPTION (43)  
1 9 Z (42) N/A  
8312300271 831214  
PDR ADPOCK 05000317

7 8 9 10  
PDR PDR  
S  
PUBLICITY  
ISSUED DESCRIPTION (45)  
[2][0][N][44] N/A  
NRC USE ONLY

NAME OF PREPARER J. F. Lohr/R. Androsik

PHONE: 301-269-4776/4986

LER NO. 83-65/3L  
DOCKET NO. 50-317  
LICENSE NO. DPR 53  
EVENT DATE 11-30-83  
REPORT DATE 12-14-83  
ATTACHMENT

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONT'D)

While troubleshooting to find the cause of the inadvertent safety injection actuation which occurred at 2243 on November 28, 1983, shortly after Unit 1 had entered Mode 3, it was determined that pressurizer pressure Channels A and B were giving higher than expected readings at this time. The source was found to be the pressure transmitters that provide the signal input to the channels. These transmitters (Barton Model 763) are currently under investigation for their failure to give proper outputs when subjected to high ambient temperature, and they will be repaired or replaced depending on the results of the investigation. In addition, monitoring procedures were initiated in August, 1982, (Ref. LER 82-40, report date August 18, 1982, and LER 82-40, Rev. 1, dated January 27, 1983) and are still in effect. These procedures include recording the four pressurizer pressure narrow range channels, two wide range channels, the pressurizer pressure test gauge installed at the sample panel, the average containment temperature and the power level.

Additionally, readings will be taken just prior to entering Mode 3 and the narrow range pressure transmitters will be recalibrated if there is an excess deviation between any two channels. During Mode 3, once a shift monitoring will be undertaken to track excess deviations and appropriate action taken.

# BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT  
CALVERT CLIFFS NUCLEAR POWER PLANT  
LUSBY, MARYLAND 20657

December 14, 1983

Dr. Thomas E. Murley  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region 1  
631 Park Avenue  
King of Prussia, PA 19406

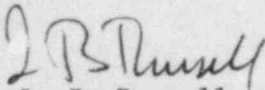
Docket No. 50-317  
License No. DPR 53

Dear Dr. Murley:

Attached is LER 83-65/3L, as required per Technical Specification 6.9.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,



L. B. Russell  
Plant Superintendent

LBR: RA:bsb

cc: Director, Office of Management Information  
and Program Control  
Messrs: A. E. Lundvall, Jr.  
J. A. Tiernan

