

NRC FORM 366
(12-81)

U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORT

APPROVED BY OMB
3150-0011
EXPIRES 0-30-82

CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 M D C C N 2 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 14 15 25 26 30 37 CAT 38

CON'T

0 1 REPORT SOURCE L 6 0 5 0 0 0 3 1 8 7 0 9 2 3 8 3 8 1 2 0 6 8 3 9
7 8 40 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 At 1715 during normal operations, 23 Auxiliary Feedwater (AFW) Pump was
0 3 found to be inoperable (T.S. 3.7.1.2.a). At 2120 the pump was returned
0 4 to service. During this period the remaining AFW pumps remained operable.
0 5 Similar events: none.
J 6
0 7
0 8

0 9 SYSTEM CODE W G 11 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE C K T B R K 14 COMP. SUBCODE F 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20
17 LER/RO REPORT NUMBER 8 3 21 EVENT YEAR 22 SEQUENTIAL REPORT NO. 0 5 2 23 OCCURRENCE CODE 0 3 24 REPORT TYPE X 25 REVISION NO. 2 26
18 ACTION TAKEN E 19 FUTURE ACTION F 20 EFFECT ON PLANT Z 21 SHUTDOWN METHOD Z 22 HOURS 0 0 0 0 23 ATTACHMENT SUBMITTED Y 24 NPRD-4 FORM SUB. N 25 PRIME COMP. SUPPLIER A 26 COMPONENT MANUFACTURER G 0 8 0
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Investigation found all #23 AFW pump motor breaker (G.E. AMH 4.76-250-
1 1 1D) protective relays tripped. The relays were reset. No operational
1 2 problems were found. Suspected cause is workers shorting internal break-
1 3 er circuits when working scaffolding while grouting at cubicle base with
1 4 door open. A facility change to add alarms is being investigated.

1 5 FACILITY STATUS E 28 % POWER 1 0 0 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY D 31 DISCOVERY DESCRIPTION NRC Inspection 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 6 ACTIVITY CONTENT RELEASE OF RELEASE Z 34 AMOUNT OF ACTIVITY N/A 35 LOCATION OF RELEASE N/A 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION N/A 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION N/A 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION N/A 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

2 0 PUBLICITY ISSUED N 44 DESCRIPTION N/A 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

NAME OF PREPARER R. J. Porter/L. F. Basso

PHONE: 301-269-4747/4933

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PDR ADOCK 05000318
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LER NO. 83-52/3X, Rev. 2
DOCKET NO. 50-318
LICENSE NO. DPR 69
EVENT DATE 09-23-83
REPORT DATE 12-06-83
ATTACHMENT

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONT)

Upon investigation, all the protective relay [redacted] motor breaker (General Electric Co. AMH 4.76-250-1 [redacted] tripped, thus the 186 lockout device was also tripped. The relays were reset.

No operational problems were revealed to account for actuation of the relays. Suspected cause is accidental shorting of internal switchgear circuits by personnel working scaffolding in the room while grouting was being done at the cubicle base with the door open.

A Facility Change is being investigated such that operators will be alerted of such out-of-service switchgear conditions. An update will be submitted upon completion of the investigation.

BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSCY, MARYLAND 20657

December 6, 1983

Dr. Thomas E. Murley
Regional Administrator
U. S. Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, PA 19406

Docket No. 50-318
License No. DPR 69

Dear Dr. Murley:

In accordance with Technical Specification 6.9, please find the attached revision to LER 83-52/3X, Rev. 2.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L. T. Russell
L. T. Russell
Plant Superintendent

LER:LFB:bsb

cc: Director, Office of Management Information
and Program Control
Messrs: A. E. Lundvall, Jr.
J. A. Tiernan

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