

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)

LICENSEE CODE

LICENSE NUMBER

LICENSE TYPE

CAT

/0/1/

REPORT

SOURCE

/L/ (6) /0/5/0/0/0/3/3/9/ (7) /0/4/1/9/8/3/ (8) /0/5/1/1/8/3/ (9)

DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On April 19, 1983, w h Unit 2 in the Refueling Mode, Fire Door S71-18, between /
/0/3/ / the 2H Emergency Diesel Room and the Turbine Building, would not latch. Since a /
/0/4/ / fire watch was immediately posted the public health and safety were not affected. /
/0/5/ / The degradation of fire doors between the Emergency Diesel Rooms and Turbine /
/0/6/ / Building due to differential pressure between the areas during diesel operation /
/0/7/ / is a recurring event. This event is contrary to T.S. 3.7.15 and reportable pur- /
/0/8/ / suant to T.S. 6.9.1.9.b. /

SYSTEM
CODECAUSE
CODECAUSE
SUBCODE

COMPONENT CODE

COMP.
SUBCODEVALVE
SUBCODE

/0/9/ /A/B/ (11) /E/ (12) /B/ (13) /X/X/X/X/X/X/ (14) /Z/ (15) /Z/ (16)

SEQUENTIAL

OCCURRENCE

REPORT

REVISION

LER/RO

EVENT YEAR

REPORT NO.

CODE

TYPE

NO.

(17)

REPORT

NUMBER

/8/3/

/-/

/0/3/4/

/ /

/0/3/

/L/

/-/

/0/

ACTION
TAKENFUTURE
ACTIONEFFECT
ON PLANTSHUTDOWN
METHOD

HOURS

ATTACHMENT
SUBMITTEDNPRD-4
FORM SUB.PRIME COMP. COMPONENT
SUPPLIER MANUFACTURER

/B/ (18) /X/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /N/ (24) /A/ (25) /C/1/7/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Fire Door S71-18 would not latch because the latch was sticking. The latch was /
/1/1/ / lubricated and operation of the fire door tested satisfactorily. Design modifi- /
/1/2/ / cation to the fire doors between the Diesel Rooms and the Turbine Building are /
/1/3/ / being pursued. /
/1/4/ /

FACILITY

METHOD OF

STATUS

ZPOWER

OTHER STATUS

DISCOVERY

DISCOVERY DESCRIPTION (32)

/1/5/ /H/ (28) /0/0/0/ (29) / NA / (30) /A/ (31) / Operator Observation /

ACTIVITY

CONTENT

RELEASED

OF RELEASE

AMOUNT OF ACTIVITY (35)

LOCATION OF RELEASE (36)

/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES

NUMBER

TYPE

DESCRIPTION (39)

/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES

NUMBER

DESCRIPTION (41)

/1/8/ /0/0/0/ (40) / NA / 8312300006 831018 /

LOSS OF OR DAMAGE TO FACILITY

(43)

PDR FOIA

GRABER83-562

PDR

/1/9/ /Z/ (42) / NA /

PUBLICITY

ISSUED

DESCRIPTION (45)

NRC USE ONLY

/2/0/ /N/ (44) / NA / / / / / / / / / / /

NAME OF PREPARER

W. R. CARTWRIGHT

PHONE

(703) 894-5151

December 20, 1982

JBW
JPC

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

LER
LER # 361-82152
EVENT DATE 821131
INFO RCVD DATE 821230 SP
GENC RCVD DATE _____

Attention: Mr. R. H. Engelken, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 82-152
San Onofre Nuclear Generating Station, Unit 2

This submittal is in accordance with the reporting requirements of Section 6.9.1.13.b of Appendix A to Facility Operating License NPF-10. It describes a reportable occurrence involving Limiting Condition for Operation (LCO) 3.3.3.6 associated with Accident Monitoring Instrumentation. A completed copy of LER 82-152 is enclosed.

While in Mode 3, at 2010 on November 21, 1982, the daily surveillance testing in accordance with procedure 5023-3-3.21.1 was in progress. Due to loss of process flow, as indicated by radiation indicator 2RI-7872-1, the Condenser Evacuation System radiation monitor (wide range) was declared inoperable. Action Statement 22 of Table 3.3-10 associated with this inoperable monitor results in an undefined action requirement due to a typographical error in the Technical Specifications. Correction of this Action Statement has been requested in License Amendment Application 15 and the action taken was as defined in that application (i.e. either restore the inoperable monitor to operable status within 7 days, or be in hot shutdown within the next 12 hours).

Subsequent investigation revealed that the monitor problem was attributable to a faulty velocity probe associated with the monitor. The faulty probe was replaced and the monitor declared operable in accordance with 5023-3-3.21.1 at 1705 on November 24, 1982. Discussions with the probe manufacturer indicate that this failure is infrequent and, therefore, this is considered an isolated event. No further corrective action is warranted at this time.

R. H. Engelken

-2-

December 20, 1982

There was no impact on plant operations or the health and safety of plant personnel or the public as a result of this event.

If there are any questions regarding the above, please contact me.

Sincerely,

HBR/TVM

H. B. RAY
STATION MANAGER

LIMayweather:VM

Enclosure: LER 82-152

cc: A. E. Chaffee (USNRC Resident Inspector, San Onofre Unit 2)

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

U. S. Nuclear Regulatory Commission
Office of Management Information and Program Control

Institute of Nuclear Power Operations