AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Statio
DATE December 15, 1983
COMPLETED BY T. P. Matthews
TELEPHONE 402-536-4733

MONTH November, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	434.6	17	457.2
2	430.8	18	457.5
3	440.9	19	457.5
4	455.3	20	457.6
5	456.8	21	458.1
6	457.3	22	458.4
7	457.1	23	458.5
8	456.8	24	459.0
9	457.4	25	459.1
10	457.7	26	459.3
11	454.4	27	459.0
12	251.4	28	458.8
13	249.5	29	458.4
14	326.5	30	458.4
15	449.9	31	
16	455.6		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO 50-285
DATE December 15, 1983
COMPLETED BY T. P. Matthews
TELEPHONE 402-536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: November, 1983
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 461
7. Maximum Dependable Capacity (Net MWe): 438
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	8,017.0	89,258.0
12. Number Of Hours Reactor Was Critical	720.0	5,759.9	69,149.9
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	720.0	5,661.0	68,658.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,029,985.8	8,034,488.0	85,651,036.4
17. Gross Electrical Energy Generated (MWH)	330,466.0	2,537,520.0	27,960,999.7
18. Net Electrical Energy Generated (MWH)	314,609.1	2,410,161.0	26,740,195.4
19. Unit Service Factor	100.0	70.6	76.9
20. Unit Availability Factor	100.0	70.6	76.9
21. Unit Capacity Factor (Using MDC Net)	99.8	68.6	65.3
22. Unit Capacity Factor (Using DER Net)	91.4	62.9	63.0
23. Unit Forced Outage Rate	0.0	0.8	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    1984 refueling outage scheduled to start around March 3, 1984.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): None

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November, 1983

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE December 15, 1983
COMPLETED BY T. P. Matthews
TELEPHONE 402-536-4733

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
									No unit shutdowns during the month of November, 1983.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending November 1983 .

1. Scheduled date for next refueling shutdown. March 1984

2. Scheduled date for restart following refueling. May 1984

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

   a. If answer is yes, what, in general, will these be?

   A Technical Specification Change

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____

   c. If no such review has taken place, when is it scheduled? _____

4. Scheduled date(s) for submitting proposed licensing action and support information. Tech. Specs. - February 1984

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:
   a) in the core 133 assemblies
   b) in the spent fuel pool 265 "
   c) spent fuel pool storage capacity 483 "
   d) planned spent fuel pool storage capacity 728 "

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by J K Gasper Date December 1, 1983

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

November, 1983
Monthly Operations Report

I. OPERATIONS SUMMARY

After completion of MTC testing at 95% power, Fort Calhoun Station returned to 100% power November 4, 1983. Power was reduced to 60% on November 12 to repair condenser tube leaks. Operation at 100% power was resumed on November 15, 1983.

During November, Fort Calhoun Station exceeded last year's record of over 200 days of continuous operation.

The NRC is onsite to give Senior Reactor Operator examinations to four engineers and two operators. Also, a Reactor Operator examination will be given to one operator.

The Spent Fuel Pool rerack continues.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Operation Incident		Deficiency
OI-1759	ST-ESF-6, F.2	During the performance of ST-ESF-6, Diesel Generator No. 2 failed to show any indication of RPM's in the control room or locally, DG-2 field flashing contactor failed to seal in, the air dampers for DG-2 failed open and the diesel stop light did not go out.
OI-1765	ST-ESF-6, F.2	During the performance of ST-ESF-6, Section F.2, Diesel Generator No. 1 tripped due to energizing of a reverse current protective relay during normal shutdown of the diesel.
OI-1780	ST-PL-1, F.2	During the performance of ST-PL-1, Section F.2, it was found that the first and second backup charging pump setpoints had drifted out of tolerance.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-FAUD-1	Fuel Assembly Uplift Condition Detection.
	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.
SP-AB-1	Hydrostatic Test of Water Side of the Auxiliary Boiler
	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it involved work on non-safety related equipment.
SP-EHC-4	EHC Coolers Leak Test
	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it involved work on non-safety related equipment.

System Acceptance Committee Packages for October, 1983:

Package	Description/Analysis
DCR 76-85	FH-1 Bridge Trolley Permissive.
	This modification allows slight movement of the bridge and trolley over the core so that fuel bundles do not hang up on each other when inserting bundles. This modification has no adverse effect on the safety analysis.
DCR 76-27	Reactor Coolant Level Indication During Cold Shutdown.
	This modification added a wide range reactor coolant level indicator which did away with the long tygon tube method. This modification has no adverse effect on the safety analysis.
DCR 74A-107	Additional Demineralized Water Outlets.
	This modification added additional demineralized water outlets in the auxiliary building. This modification has no adverse effect on the safety analysis.

Package	Description/Analysis
EEAR FC-79-59	Outlet for Jib Crane at Loading Dock. This modification added a 480V outlet to the truck dock only. This did not involve any safety related equipment. This modification has no adverse effect on the safety analysis.
DCR 77-64	Fort Calhoun Station Undervoltage Protection. This modification resulted in improvement in the undervoltage protection. This modification has no adverse effect on the safety analysis.
EEAR FC-80-136	Minor Electrical Modifications. This modification consisted of adding minor power receptacles and lighting. This did not involve any safety related equipment. This modification has no adverse effect on the safety analysis.
EEAR FC-80-119	Emergency Lighting. This modification added emergency lighting in the stairwells leading down to the maintenance shop. This did not involve any safety related equipment. This modification has no adverse effect on the safety analysis.
EEAR FC-83-112	Turbine Building Crane Catwalk. This modification resulted in the installation of a catwalk for the turbine building crane. This did not involve any safety related equipment. This modification has no adverse effect on the safety analysis.
EEAR FC-81-149 Phase II	FW-8 Corrosion This modification was done to install new seal water heat exchangers and separator tanks onto the condenser evacuation pumps FW-8A,B and C. This did not involve any safety related equipment. This modification has no adverse effect on the safety analysis.

Package	Description/Analysis
EEAR FC-81-140	TAR Recorders 1, 2, 3 and 4 Replacement. This modification ensures heat tracing points are read and recorded daily to avoid Technical Specification violations. This modification has no adverse effect the safety analysis.
EEAR FC-83-118	Discharge Line Off Chemical Lagoon Recirculation Pump. This modification did not involve a safety related system. This modification has no adverse effect on the safety analysis.
DCR 77-83	Expansion of P-250 Computer Capabilities. This modification added a second computer only. This modification has no adverse effect on the safety analysis.
EEAR FC-80-132	Rework of Chemistry Lab Fume Hoods. This modification did not involve a safety related system. This modification has no adverse effect on the safety analysis.
DCR 75B-24	Instrument Inverter Frequency Detection. This modification improved the safe operation of the clutch coils by keeping the instrument inverters in synchronization and not tripping them out spuriously. This modification has no adverse effect on the safety analysis.
DCR 74A-36	RPS Cable Rerouting. This modification only changed the location of RPS cableing and did not effect the operation of the cableing. This modification has no adverse effect on the safety analysis.
EEAR FC-83-110	Modification to Upgrade QA Records File Room. This modification upgraded the fire protection qualities of the QA Records File Room. This modification has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

There are no new leak rate results to report.

F. CHANGES IN PLANT OPERATING STAFF

None

G. TRAINING

Training for November was conducted as scheduled in the areas of operator requalification, non-licensed operators, fire brigade, maintenance and general employee training. Additional training was conducted for seven NRC licensed candidates who were examined the week of November 28 by the Commission.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None

II. MAINTENANCE (Significant Safety Related)

M.O. No.	DATE	DESCRIPTION	CORRECTIVE ACTION
22086	10/31/83	Pressurizer backup heaters bank No. 3 groups No. 9 will not energize.	Per PRC approved procedure.
21878	10/14/83	Loop flow for RC-2A loop failed high.	Replaced transmiter, calibrated per CP-RPS-3.

W. Gary Gates

W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

December 15, 1983
LIC-83-316

Mr. Richard C. DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Reference: Docket No. 50-285

Dear Mr. DeYoung:

Please find enclosed ten (10) copies of the November Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

W. C. Jones
Division Manager
Production Operations

WCJ/TPM:jmm

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
NRC File

IE24
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