



GPU Nuclear Corporation  
Post Office Box 480  
Route 441 South  
Middletown, Pennsylvania 17057-0191  
717 944-7621  
TELEX 84-2386  
Writer's Direct Dial Number:

December 16, 1983  
5211-83-365

Office of Management Information  
and Program Control  
c/o Distribution Services Branch  
DPC, ADM  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit 1 (TMI-1)  
Operating License No. DPR-50  
Docket No. 50-289  
Monthly Operating Report for November

Enclosed please find two (2) copies of the November Operating Report for  
Three Mile Island Nuclear Station, Unit 1.

Sincerely,

*J. J. Coley for H. D. Hukill*  
H. D. Hukill  
Director, TMI-1

HDH:vjf

Enclosures

cc: V. Stello  
Dr. Thomas E. Murley

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# OPERATING DATA REPORT

DOCKET NO. 50-289  
 DATE December 16, 1983  
 COMPLETED BY C. W. Smyth  
 TELEPHONE (717) 948-8551

## OPERATING STATUS

		NOTES
1. UNIT NAME:	THREE MILE ISLAND UNIT 1	
2. REPORTING PERIOD:	NOVEMBER, 1983.	
3. LICENSED THERMAL POWER (MWT):	2535.	
4. NAMEPLATE RATING (GROSS MWE):	871.	
5. DESIGN ELECTRICAL RATING (NET MWE):	819.	
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE):	824.	
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE):	776.	

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: \_\_\_\_\_
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE) \_\_\_\_\_
10. REASONS FOR RESTRICTIONS, IF ANY: \_\_\_\_\_
- \_\_\_\_\_
- \_\_\_\_\_

	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	720.	8016.	81049.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	31731.8
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	838.5
14. HOURS GENERATOR ON-LINE	0.0	0.0	31180.9
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.	0.	76531071.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	25484330.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	23840053.
19. UNIT SERVICE FACTOR	0.0	0.0	38.5
20. UNIT AVAILABILITY FACTOR	0.0	0.0	38.5
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	0.0	37.6
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	0.0	35.9
23. UNIT FORCED OUTAGE RATE	100.0	100.0	57.7

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH  
 \_\_\_\_\_  
 \_\_\_\_\_
25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289  
UNIT TMI-1  
DATE December 16, 1983  
COMPLETED BY C. W. Smyth  
TELEPHONE (717) 948-8551

MONTH: NOVEMBER

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0.	17	0.
2	0.	18	0.
3	0.	19	0.
4	0.	20	0.
5	0.	21	0.
6	0.	22	0.
7	0.	23	0.
8	0.	24	0.
9	0.	25	0.
10	0.	26	0.
11	0.	27	0.
12	0.	28	0.
13	0.	29	0.
14	0.	30	0.
15	0.	31	N/A
16	0.		

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November, 1983

DOCKET NO. 50-289  
UNIT NAME TMI-I  
DATE December 16, 1983  
COMPLETED BY C. W. Smyth  
TELEPHONE (717) 948-8551

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	83/11/01	F	744	D	1	N/A	ZZ	ZZZZZZ	Regulatory Restraint Order

<sup>1</sup>  
F - Forced  
S - Scheduled

<sup>2</sup>  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup>  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Other (Explain)

<sup>4</sup>  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File (NUREG-  
0161)

<sup>5</sup>  
Exhibit I - Same Source

## OPERATIONS SUMMARY

The Plant has been in cold shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System. The Primary System was partially drained for the repair of DH-V1. It was maintained partially drained until 11/24. The root valves for the Reactor Vessel water level trending instrumentation and the head vents were installed. The Primary System was then re-filled for the remainder of the month and was maintained with a bubble in the pressurizer and RCS pressure at about 310 psig and a temperature of 120-130°F.

Upon filling the Primary System, sulfates and chlorides exhibited a definite increasing trend. One RCP was run continuously to prevent the settling out and concentration of the contaminants while cleanup was in progress. By the end of the month, both chlorides and sulfates had been reduced to well within specifications.

## MAJOR SAFETY RELATED MAINTENANCE

The following work was performed during the month:

### Snubber Testing

The Snubber Testing and Repair program continued with approximately 20 snubbers being rebuilt. This program is nearly completed and has resulted in the overhaul of all but about 20 snubbers in the plant.

### DH-V1 Repair

Decay Heat Removal valve DH-V1 (RCS Loop B DHR suction valve) had been disassembled the previous month after it was found to be leaking (body to bonnet seal ring leak). This valve had been furmanited approximately six years ago. Minor erosion of the seal ring was found. The seal ring was replaced and the valve reassembled.

### Core Flood Valve Repair

The following work was performed on three core flood system valves:

- CF-V1A (14" "A" CFT Dischg Iso Valve) - repaired pressure seal ring leak
- CF-V1B (14" "B" CFT Dischg Iso Valve) - repacked
- CF-V4B (14" "B" CFT Dischg Check Valve) - repaired pressure seal ring leak.

### Main Steam Safety Relief Valves

A major program to disassemble, inspect, and perform seat work on 14 main steam safeties was started. Nine of the 14 were completed. The remaining valves will be completed during December. This work is being performed because the 14 valves were found to be weeping during Hot Functional Testing.

WDG-V-12

Waste Gas System relief valve V-12 was found to be leaking through and was disassembled, overhauled and reassembled. The valve was satisfactorily bench tested and reinstalled.

MU-V16A

Makeup System valve MU-V16A (HPI Isolation Valve) was disassembled for replacement of a scored stem.

Valve Repacking Program

A major program of valve repacking was completed during the month with numerous valves in the Reactor Coolant System, Decay Heat Removal System, and the Makeup and Purification System being repacked.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).