

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

| | | | | | | | | | | | | | | | | | | | | | | | | | |
|---|---|---|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|
| 0 | 1 | M | I | P | A | L | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5 |
| 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 | 17 | 18 | 19 | 20 | 21 | 22 | 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 |

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|---|---|---|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|---|
| 0 | 1 | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 5 | 5 | 7 | 1 | 1 | 0 | 9 | 8 | 3 | 8 | 1 | 2 | 0 | 9 | 8 | 3 | 9 |
| 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 | 17 | 18 | 19 | 20 | 21 | 22 | 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | |

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 While shutdown for refueling, a routine refueling duration surveillance test

0 3 revealed that fourteen of twenty-four main steam relief valves lifted at

0 4 pressures in excess of the upper limit specified in TS 3.1.7.c. The valves

0 5 are believed to have been able to provide their intended safety function,

0 6 however, evaluation is incomplete at this time. Notification will be pro-

0 7 vided if final analysis is to the contrary. Reportable per TS 6.9.2.b(1).

0 8 (The valves have been disassembled, rebuilt and successfully re-tested.)

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|---|---|---|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|
| 0 | 9 | C | C | 11 | X | 12 | Z | 13 | V | A | L | V | E | X | 14 | H | 15 | B | 16 | 8 | 3 | 0 | 7 | 4 | 0 | 3 | L | 0 | 0 | 7 | 1 | 0 | 26 |
| 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 | 17 | 18 | 19 | 20 | 21 | 22 | 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 33 | 34 | 35 | 36 | 37 | 38 | 39 | 40 |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Cause is believed to have resulted from adjustments made to each valve's

1 1 blowdown ring which affected the lift setting. These adjustments were made

1 2 subsequent to the final lift setting adjustments made during the 1981

1 3 Refueling Outage. The test procedure will be revised to prevent blowdown

1 4 adjustments prior to "as found" testing, and subsequent to "as left" testing.

| | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
|---|---|---|----|----|----|----|----|----|----|----|----|-------------------|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|----|
| 1 | 5 | H | 28 | 0 | 0 | 0 | 29 | NA | 30 | B | 31 | Surveillance Test | 32 | NA | 36 | 0 | 0 | 0 | 37 | Z | 38 | NA | 39 | 0 | 0 | 0 | 40 | NA | 41 | 0 | 0 | 0 | 42 | NA | 43 | 0 | 0 | 0 | 44 | NA | 45 | NA | 46 | NA | 47 | NA | 48 | NA | 49 | NA | 50 | NA | 51 | NA | 52 | NA | 53 | NA | 54 | NA | 55 | NA | 56 | NA | 57 | NA | 58 | NA | 59 | NA | 60 | NA | 61 | NA | 62 | NA | 63 | NA | 64 | NA | 65 | NA | 66 | NA | 67 | NA | 68 | NA | 69 | NA | 70 | NA | 71 | NA | 72 | NA | 73 | NA | 74 | NA | 75 | NA | 76 | NA | 77 | NA | 78 | NA | 79 | NA | 80 | NA |
| 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 | 17 | 18 | 19 | 20 | 21 | 22 | 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 33 | 34 | 35 | 36 | 37 | 38 | 39 | 40 | 41 | 42 | 43 | 44 | 45 | 46 | 47 | 48 | 49 | 50 | 51 | 52 | 53 | 54 | 55 | 56 | 57 | 58 | 59 | 60 | 61 | 62 | 63 | 64 | 65 | 66 | 67 | 68 | 69 | 70 | 71 | 72 | 73 | 74 | 75 | 76 | 77 | 78 | 79 | 80 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |

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JGKeppler, Administrator
Palisades Plant
LER 83-74
December 9, 1983

During the current refueling outage, all twenty-four (24) main steam relief valves were removed and sent off-site for surveillance testing per Surveillance Procedure RM-29. The results of the "as found" testing indicate that fourteen (14) valves lifted at test pressures in excess of the upper limit specified in TS 3.1.7.c. The maximum pressure at which a valve lifted was 1147 psig and the minimum pressure in excess of the TS limit was 1040 psig.

As described in FSAR Chapter 4.3.4, the design at Palisades is such that the main steam relief valves are divided into three groups, each group having a different lift setting. The performance of the surveillance test resulted in a total of twenty-three (23) relief valves lifting at pressures in excess of the upper limits of the FSAR and surveillance test acceptance criteria for each group. Fourteen (14) of these valves were also in excess of the upper limit imposed by TS 3.1.7.c as described above.

The cause is believed to have resulted from adjustments made during the previous refueling outage to the valve's blowdown ring and subsequent to the final lift setting adjustment which was made at that time. Adjustments to the valve's blowdown ring were permitted by procedure. Adjustments to the blowdown ring were not anticipated to have an effect on the lift pressure value.

The test procedure will be revised to prohibit any adjustments to the relief valve blowdown ring prior to "as found" testing or subsequent to "as left" testing.



Consumers
Power
Company

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December 9, 1983

James G Keppler, Administrator
Region III
US Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 83-74 (VALVES FAILED TO MEET "AS
FOUND" ACCEPTANCE CRITERIA)

Attached please find Licensee Event Report 83-74 (Valves Failed To Meet "As
Found" Acceptance Criteria) which is reportable to the NRC per Technical
Specification 6.9.2.b(1).

Brian D Johnson
Staff Licensing Engineer

CC Director, Office of Nuclear Reactor Regulation
Director, Office of Inspection and Enforcement
NRC Resident Inspector - Palisades

Attachments

DEC 14 1983

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