

## LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 N Y N M P 1 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5

LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 37 CAT 38

CON'T 01 L 6 0 5 0 0 0 2 2 0 7 1 1 1 0 8 3 8 1 2 0 9 8 3 9

REPORT SOURCE 60 61 DOCKET NUMBER 63 65 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

012 During normal plant operation, an NRC resident inspector discovered that

013 since the plant startup in June, 1983 the Emergency Condenser Isolation

014 Valves have not been stroke time-tested correctly. According to Tech Spec

015 4.2.7a and Table 3.2.7 these valves are to be timed from the fully opened

016 to fully closed position. The test procedure called for them to be tested

017 in the opposite direction. These valves are normally closed and upon system

018 initiation the valve is required to open, when in the open position they are

019

SYSTEM CODE C E 11 CAUSE CODE D 12 CAUSE SUBCODE Z 13 COMPONENT CODE V A L V E X 14 COMP. SUBCODE D 15 VALVE SUBCODE D 16

LER-RO REPORT NUMBER 17 EVENT YEAR 8 3 SEQUENTIAL REPORT NO. 0 3 8 OCCURRENCE CODE 0 3 REPORT TYPE L REVISION NO. 0

ACTION TAKEN G 18 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 ATTACHMENT SUBMITTED Y 23 NRC-4 FORM SUB. N 24 PRIME COMP. SUPPLIER A 25 COMPONENT MANUFACTURER A 5 8 5

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

110 Emergency Condenser isolation valve 39-06 was tested in both directions the

111 corresponding valve in emergency condenser loop #11 (39-05) had been tested

112 after the performance of maintenance work during the last outage, hence did

113 not need to be retested. The test procedure has been revised to comply with

114 Tech Spec 4.2.7a. The test was completed and Emergency Condenser Loop #12

115

FACILITY STATUS E 28 % POWER 1 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY D 31 DISCOVERY DESCRIPTION By Resident NRC Inspector 32

ACTIVITY CONTENT Z 33 RELEASED OF RELEASE Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36

PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39

PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41

LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43

PUBLICITY ISSUED N 44 DESCRIPTION NA 45

NAME OF PREPAPER J.L. Raby PHONE: (315) 349-2620

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PDR ADDCK 05000220  
S PDR

APN-21 -18

January 1978

LER 83-38 Continued

Event Description and Probable Consequences (continued)

required to close upon an isolation signal, therefore, they should be tested in both directions. In accordance with Tech Spec 3.1.3c Emergency Condenser Loop #12 was taken out of service so that the Emergency Condenser isolation valve No. 39-06 could be tested. The corresponding valve in emergency condenser loop #11 (39-05) had been tested after the performance of maintenance work during the last outage, hence did not need to be retested.

Cause Description and Corrective Actions (continued)

was put back into service within the seven day period allowed by Tech Spec 3.1.3c. A complete review of technical specification surveillance requirements and surveillance test procedures will be performed to assure that all requirements are being met. This review will be performed by an independent organization on an expedited basis.

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

300 ERIE BOULEVARD WEST  
SYRACUSE, N.Y. 13202

THOMAS E. LEMPGES  
VICE PRESIDENT—NUCLEAR GENERATION

December 12, 1983

Dr. Thomas E. Murley  
Regional Administrator  
United States Nuclear Regulatory Commission  
Region I  
631 Park Avenue  
King of Prussia, PA 19406

Re: Docket No. 50-220  
LER 83-38

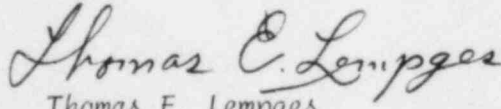
Dear Dr. Murley:

In accordance with Nine Mile Point Nuclear Station Unit #1  
Technical Specifications, we hereby submit the following Licensee  
Event Report:

LER 83-38      Which is being submitted in accordance with  
Section 6.9.2(b)2 conditions leading to  
operation in a degraded mode permitted by  
a limiting condition for operation or plant  
shutdown required by a limiting condition  
for operation.

This report was completed in the format designated in  
NUREG-0161, dated July 1977.

Very truly yours,

  
Thomas E. Lempges

TEL/JLR/mtm  
Attachments (3 copies)  
cc: Director, Office of I&E (30 copies)  
Director, Office of MIPC (3 copies)

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