

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

NOVEMBER 1983

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The condenser cooling method is a closed cycle spray canal, and the Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively, pursuant to Docket Numbers 50-254 and 50-265. The date of initial reactor criticalities for Units 1 and 2 respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit 1 and March 10, 1973 for Unit 2.

This report was compiled by Becky Brown and Alex Misak, telephone number 309-654-2241, extensions 127 and 194.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

November 1-4: Unit One began the month operating at full power and maintained this load until 1914 hours on November 1, when the unit was manually scrammed due to the loss of the Turbine Building Closed Cooling Water System caused by a valving error. At 0440 hours on November 2, the Reactor became critical and remained critical until 0900 hours when the Reactor scrammed due to a loss of pressure in the Scram Air Header caused by a valving error. The Reactor again became critical at 1425 hours, and the unit was placed on line at 1825 hours, at which time a normal load increase to full power was initiated.

November 5-10: At 0050 hours on November 5, the unit dropped load from 800 MWe to 600 MWe due to low Off Gas flow caused by Off Gas combustion in the Steam Jet Air Ejector. At 0315 hours, the combustion was stopped by closing the primary jet suction to the 'A' Steam Jet Air Ejector, and a normal load increase was initiated. At 0930 hours, the unit was again forced to drop load to 600 MWe due to high Main Condenser backpressure caused by operation with the 'A' Steam Jet Air Ejector primary jet suction closed to prevent Off Gas combustion. The unit held load at 600 MWe until 1715 hours when a normal load increase was initiated. At 2000 hours, on November 7, the unit dropped load from 810 MWe to 650 MWe to backwash a Condensate Demineralizer. At 2230 hours, backwashing was completed, and a normal load increase was initiated.

November 11-19: At 2200 hours on November 11, the unit dropped load from 815 MWe to 550 MWe to change the Control Rod Pattern. The Control Rod moves were completed by 0230 hours on November 12, and a normal load increase was initiated. At 0915 hours on November 18, the unit dropped load from 815 MWe to 580 MWe due to Main Condenser vacuum problems caused by a Steam Jet Air Ejector Relief Valve lifting. At 1015 hours, the valve was repaired and the unit began a normal load increase.

November 20-30: At 0030 hours on November 20, the unit dropped load from 810 MWe to 700 MWe to perform weekly Turbine tests. At 0200 hours, the tests were completed, and a normal load increase was initiated. At 0035 hours on November 27, the unit dropped load from 810 MWe to 700 MWe to perform weekly Turbine tests. The tests were completed at 0300 hours, and a normal load increase was initiated. At 2100 hours on November 29, the unit dropped load from 820 MWe to 700 MWe to switch Condensate pumps. At 2230 hours, the pump switch was completed, and the unit began a normal load increase.

B. Unit Two

Unit Two remained shutdown throughout the month for End of Cycle Six Refueling and Maintenance.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY
RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Unit 1 and 2 during the reporting period. This summary includes the following headings: Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q28222		1/2 Diesel Generator - Fuel Prime Pump Motor	The commutator was dirty.	The fuel prime pump motor would not run. This did not make the Diesel Generator inoperable.	The commutator was cleaned and the coupling was adjusted. Pump operation was then verified.
Q29782		Shock Suppressor PSA#3 Hanger No. M986-D97, on SBGTS line 7509- 24" (SN 25170)	10 CFR 21 notification of possible capstan spring degrada- tion.	Both SBGTS were inoperable during snubber replacement.	The snubber was replaced.

UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q28849		250 Volt Battery Cell #76	The cell failed the discharge test.	The whole 250 Volt Battery still passed the discharge test even with Cell #76 failure.	The cell was replaced.
Q29128		2-599-24A ACAD Torus Injection Check Valve	Valve internals were fouled, preventing good seating.	Valve leak 15.1 scfh at 48 psig pressure. After repairs it leaked 0.0 scfh.	Valve internals were cleaned and its flex gasket was replaced. The valve was then successfully leak rate tested.
Q29170		Sensor Removed From 'B' Fuel Pool ARM	Input transistors Q ₁ & Q ₂ on the main amplifier board were failed.	The sensor had been removed from the ARM and replaced with a Red Tagged Spare on Work Request Q29149.	Transistors Q ₁ & Q ₂ were replaced & the sensor was source calibrated, and replaced in the ARM.
Q29788		Reactor Bldg Fuel Pool Channel 'B' Monitor	The sensor failed.	The monitor read 10 ⁵ mr/hr. Radiation Protection readings indicated normal levels.	The sensor was replaced with a new calibrated sensor. The monitor was functionally tested.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

<u>UNIT ONE</u>		
<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
83-45/01T	11-21-83	Generic Problem with Namco Limit Switches
83-46/03L	11-28-83	1C RHR Service Water Pump Out of Service
<u>UNIT TWO</u>		
83-19/03L	11-07-83	2A 24/48 Volt Battery Failed Discharge Test
83-22/01T	11-15-83	Both MSIV Room Doors Open; Loss of Secondary Containment
83-23/03L	11-28-83	Main Steamline Hi-Flow Switch Setpoint Drift

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATED December 1

COMPLETED BY Alex Misak

TELEPHONE 309-654-2241x194

OPERATING STATUS

0000 110183

1. Reporting period: 2400 113083 Gross hours in reporting period: 720

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>705.2</u>	<u>7640.4</u>	<u>82811.6</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>696.8</u>	<u>7517.2</u>	<u>79603.9</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. Gross thermal energy generated (MWH)	<u>1644743</u>	<u>17087388</u>	<u>163300379</u>
10. Gross electrical energy generated (MWH)	<u>538517</u>	<u>5539670</u>	<u>52661551</u>
11. Net electrical energy generated (MWH)	<u>508793</u>	<u>5210811</u>	<u>49039719</u>
12. Reactor service factor	<u>97.9</u>	<u>95.3</u>	<u>81.7</u>
13. Reactor availability factor	<u>97.9</u>	<u>95.3</u>	<u>85.1</u>
14. Unit service factor	<u>96.8</u>	<u>93.8</u>	<u>78.6</u>
15. Unit availability factor	<u>96.8</u>	<u>93.8</u>	<u>79.5</u>
16. Unit capacity factor (Using MDC)	<u>91.4</u>	<u>84.4</u>	<u>62.9</u>
17. Unit capacity factor (Using Des. MWe)	<u>89.1</u>	<u>82.2</u>	<u>61.3</u>
18. Unit forced outage rate	<u>3.2</u>	<u>2.2</u>	<u>6.3</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

#UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE December 1

COMPLETED BY Alex Misak

TELEPHONE 309-654-2241x194

OPERATING STATUS

0000 110183

1. Reporting period: 2400 113083 Gross hours in reporting period: 720

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>0.0</u>	<u>5654.1</u>	<u>77917.5</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>0.0</u>	<u>5621.7</u>	<u>75209.8</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated (MWH)	<u>0</u>	<u>10790594</u>	<u>155382088</u>
10. Gross electrical energy generated (MWH)	<u>0</u>	<u>3398245</u>	<u>49435780</u>
11. Net electrical energy generated (MWH)	<u>-2494</u>	<u>3151329</u>	<u>46334896</u>
12. Reactor service factor	<u>0.0</u>	<u>70.5</u>	<u>77.6</u>
13. Reactor availability factor	<u>0.0</u>	<u>70.5</u>	<u>80.6</u>
14. Unit service factor	<u>0.0</u>	<u>70.1</u>	<u>74.9</u>
15. Unit availability factor	<u>0.0</u>	<u>70.1</u>	<u>75.6</u>
16. Unit capacity factor (Using MDC)	<u>-0.5</u>	<u>51.2</u>	<u>60.0</u>
17. Unit capacity factor (Using Des. MWe)	<u>-0.5</u>	<u>49.9</u>	<u>58.5</u>
18. Unit forced outage rate	<u>0.0</u>	<u>1.8</u>	<u>8.6</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup 1-19-84

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

POCKET NO. 50-254

UNIT ONE

DATED December 1

COMPLETED BY Alex Misak

TELEPHONE 309-654-2241x194

MONTH November 1983

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>594.1</u>
2.	<u>54.3</u>
3.	<u>549.9</u>
4.	<u>699.7</u>
5.	<u>633.1</u>
6.	<u>734.2</u>
7.	<u>736.8</u>
8.	<u>752.1</u>
9.	<u>763.1</u>
10.	<u>762.9</u>
11.	<u>751.0</u>
12.	<u>572.4</u>
13.	<u>661.7</u>
14.	<u>681.1</u>
15.	<u>682.6</u>
16.	<u>843.3</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>763.1</u>
18.	<u>712.4</u>
19.	<u>755.7</u>
20.	<u>752.9</u>
21.	<u>749.7</u>
22.	<u>778.3</u>
23.	<u>765.2</u>
24.	<u>769.5</u>
25.	<u>764.2</u>
26.	<u>764.6</u>
27.	<u>750.2</u>
28.	<u>765.1</u>
29.	<u>765.2</u>
30.	<u>763.0</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATED December 1

COMPLETED BY Alex Misak

TELEPHONE 309-654-2241x194

MONTH November 1983

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>-3.5</u>
2.	<u>-3.4</u>
3.	<u>-3.4</u>
4.	<u>-3.3</u>
5.	<u>-3.3</u>
6.	<u>-3.5</u>
7.	<u>-3.5</u>
8.	<u>-3.3</u>
9.	<u>-3.2</u>
10.	<u>-3.7</u>
11.	<u>-4.7</u>
12.	<u>-5.0</u>
13.	<u>-5.0</u>
14.	<u>-5.0</u>
15.	<u>-4.9</u>
16.	<u>-4.9</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>-4.8</u>
18.	<u>-3.8</u>
19.	<u>-4.9</u>
20.	<u>-4.9</u>
21.	<u>-5.0</u>
22.	<u>-4.5</u>
23.	<u>-4.3</u>
24.	<u>-4.5</u>
25.	<u>-4.2</u>
26.	<u>-4.3</u>
27.	<u>-4.4</u>
28.	<u>-4.5</u>
29.	<u>-4.7</u>
30.	<u>-2.5</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit One

COMPLETED BY A. Misak

DATE December 5, 1983

REPORT MONTH NOVEMBER 1983

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
83-76	831101	F	0.0	G	5		WG	HTEXCH	Reduced load due to Turbine Building Closed Cooling Water System Low Pressure
83-77	831101	F	13.8	G	2		WG	HTEXCH	Reactor scrammed due to loss of Turbine Building Closed Cooling Water System
83-78	831102	F	9.4	G	3		RB	ZZZZZZ	Reactor scrammed due to low scram air header pressure
83-79	831105	F	0.0	B	5		HC	ZZZZZZ	Reduced load due to low Off Gas Flow
83-80	831105	F	0.0	H	5		HC	HTEXCH	Reduced load due to high Condenser backpressure
83-81	831107	F	0.0	B	5		HG	DEMINX	Reduced load to backwash Condensate Demineralizers
83-82	831111	S	0.0	B	5		RB	CONROD	Reduced load to change Control Rod Pattern
83-83	831118	F	0.0	H	5		HC	HTEXCH	Reduced load due to Main Condenser Vacuum problems
83-84	831120	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine tests

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AUG 16 1982

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13

Revision 6

August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit One

COMPLETED BY A. Misak

DATE December 5, 1983

REPORT MONTH NOVEMBER 1983

TELEPHONE

309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
33-85	831127	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine tests
83-86	831129	S	0.0	B	5		HH	PUMPXX	Reduced load for Condensate Pump Switch
APPROVED AUG 16 1982									

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VCUSK

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONSQTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265

UNIT NAME Quad-Cities Unit Two

COMPLETED BY A. Misak

DATE December 5, 1983

REPORT MONTH NOVEMBER 1983

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
83-66	830904	S	720.0	C	4		RC	FUELXX	Unit Two remains shutdown for End of Cycle Six Refueling and Maintenance

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VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. MAIN STEAM RELIEF VALVE OPERATIONS

There were no Main Steam Relief Valve Operations for the reporting period.

B. CONTROL ROD DRIVE SCRAM TIMING DATA FOR UNITS ONE AND TWO

There was no Control Rod Drive Scram Timing Data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: Q1 Reload: 6 Cycle: 7
2. Scheduled date for next refueling shutdown: 9-6-82
3. Scheduled date for restart following refueling: 12-18-82
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: Yes
5. Scheduled date(s) for submitting proposed licensing action and supporting information: 8-19-82: Tech. Spec. changes submitted to the NRC.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
- a) All 7x7 fuel assemblies will be removed from the core.
 - b) MAPLHGR curves for fuel types in the core are being extended to 40,000 MWD/ST.
 - c) MCPR limits will be determined by GE's ODYN computer code.
 - d) The vessel pressure safety limit is being modified to accommodate the potential for higher reactor pressures as calculated by ODYN.
7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 1730
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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APR 20 1978

Q. C. O. S. R.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: Q2 Reload: 6 Cycle: 7
2. Scheduled date for next refueling shutdown: 9-5-83
3. Scheduled date for restart following refueling: 11-12-83
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
No, however, a change to the Technical Specifications is being submitted (see below).
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
June 14, 1983 (Scheduled)
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
a) All new fuel assemblies will be of barrier design; MAPLHGR curves will be re-labeled to include the barrier designation.
b) The use of improved assumptions in the load reject without bypass analysis resulted in a much improved MCPR operating limit. Technical Specifications are being changed to provide this additional operating margin.
7. The number of fuel assemblies.
a. Number of assemblies in core: 0
b. Number of assemblies in spent fuel pool: 1136
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
a. Licensed storage capacity for spent fuel: 3897
b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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Q. C. O. S. R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center



Commonwealth Edison

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NJK-83-446

December 1, 1983

Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555
Attention: Document Control Desk

Gentlemen:

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of November 1983.

Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

bb

Enclosure

IE2A
11