

LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N C B E P 1 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5

CON'T
0 1 L 6 0 5 0 - 0 3 2 5 7 1 1 1 0 8 3 8 1 2 1 3 8 3 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0 2 During unit shutdown operations, performance of Reactor High Pressure Trip Response Time Test, PT-A26.2,
0 3 revealed that reactor vessel steam dome pressure high instrument, B21-PT-N023D, actuated out of specifi-
0 4 cations (0.59 seconds versus the technical specification requirement of < 0.55 seconds), due to an incorrect
0 5 instrument damping adjustment. This instrument, which is response time tested every 72 months, was placed
0 6 in service on June, 27, 1981. This event did not affect the health and safety of the public.

0 7

0 8 Technical Specifications 3.3.1, 6.9.1.9a

0 9 I A 11 A 12 X 13 I N S T R U 14 T 15 Z 16
 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
17 8 3 0 5 8 0 3 L 0
 LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.
 ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NRPD-6 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
X 18 Z 19 Z 20 Z 21 0 0 0 0 Y 23 Y 24 N 25 R 3 6 9 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 The incorrect instrument damping adjustment is believed to have been made by the instrument manufacturer.
1 1 The damping adjustment setting was readjusted and the N023D, Model No. 1152, tested satisfactorily.
1 2 The respective instrument damping adjustments of 75 Unit No. 1, 52 Unit No. 2, and 44 in plant stock
1 3 (Model No. 1152) instruments were checked. One of these instruments, 2-B21-LT-N042A, was found to have
1 4 improperly positioned dampening adjustments.

1 5 G 28 0 0 0 29 NA 30 B 31 Periodic Test
 FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION
1 6 Z 33 Z 34 NA NA 35 NA 36
 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE
1 7 0 0 0 37 Z 38 39
 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION
1 8 0 0 0 40 41
 PERSONNEL INJURIES NUMBER DESCRIPTION
1 9 Z 42 43
 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION
2 0 N 44 45
 PUBLICITY ISSUED DESCRIPTION
 NAME OF PREPARER R. M. Poulk, Jr. PHONE: 919-457-9521

NA
 8312200104 831213
 PDR ADDCK 05000325
 S PDR

FE22

NRC USE ONLY

LER ATTACHMENT - RO # 1-83-58

Facility: BSEP Unit No. 1

Event Date: November 10, 1983

B21-PT-N023D is one of four pressure transmitters which initiate a Reactor Protection System (RPS) reactor scram on reactor pressure vessel high pressure. These four instruments, Rosemount Model Number 1152, are designed in a one-out-of-two-taken-twice logic ([N023A or C] and [N023P or D]). Technical specifications require each of these instruments to actuate in less than or equal to 0.55 seconds.

This event was discovered during a unit shutdown while performing a response time test of the N023D instrument in accordance with Reactor High Pressure Trip Response Time Test, PT-A26.2. It was found that N023D actuated at 0.59 seconds. Investigation of this discovery determined the instrument would not actuate within specifications because the instrument damping adjustment was at the midpoint setting. The required adjustment for this instrument is the minimum setting. Incorrect setting of this instrument could have allowed the RPS logic response to exceed its analyzed times and thus potentially exceed the bounding accident parameters.

The instrument manufacturer's literature specifies that this model instrument is calibrated and shipped with the instrument damping adjustment set at the counterclockwise stop which is the minimum adjustment setting. When this discovery was made, the adjustment was found with a factory seal; therefore, incorrect setting is attributed to the instrument's manufacturer. The N023D was properly adjusted and tested satisfactorily in accordance with the PT.

Vendor instructions stated that the setting was set to minimum at the factory and response times were evaluated on this basis. Based on this information, no testing was done to ensure the adjustment was set to minimum. The requirements for response time testing was evaluated by the Plant Nuclear Safety Committee on July 26, 1982, as per Engineering Evaluation Report EER-82-012.

Following this discovery, a check of the respective instrument damping adjustment settings of 75 Unit No. 1, 52 Unit No. 2, and 44 plant stock (Model No. 1152) instruments was performed. These instruments are used throughout the plant in RPS, ECCS, and PCIV actuation instrumentation. This check required breaking the factory seal and attempting to turn the damping adjustment in the counterclockwise direction. This check determined that one additional instrument (2-B21-LT-N042A) had a dampering adjustment not fully counterclockwise; however, as response time requirements do not exist for this instrument, it was not considered to be a safety problem.



Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
December 13, 1983

FILE: B09-13510C
SERIAL: BSEP/83-3944

Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-83-58

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9a of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977. This Licensee Event Report is being submitted three days late as discussed with the Senior Resident Inspector.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/clh/LETC1

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

OFFICIAL COPY

1622

11