

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100 101 102 103 104 105 106 107 108 109 110 111 112 113 114 115 116 117 118 119 120 121 122 123 124 125 126 127 128 129 130 131 132 133 134 135 136 137 138 139 140 141 142 143 144 145 146 147 148 149 150 151 152 153 154 155 156 157 158 159 160 161 162 163 164 165 166 167 168 169 170 171 172 173 174 175 176 177 178 179 180 181 182 183 184 185 186 187 188 189 190 191 192 193 194 195 196 197 198 199 200 201 202 203 204 205 206 207 208 209 210 211 212 213 214 215 216 217 218 219 220 221 222 223 224 225 226 227 228 229 230 231 232 233 234 235 236 237 238 239 240 241 242 243 244 245 246 247 248 249 250 251 252 253 254 255 256 257 258 259 260 261 262 263 264 265 266 267 268 269 270 271 272 273 274 275 276 277 278 279 280 281 282 283 284 285 286 287 288 289 290 291 292 293 294 295 296 297 298 299 300 301 302 303 304 305 306 307 308 309 310 311 312 313 314 315 316 317 318 319 320 321 322 323 324 325 326 327 328 329 330 331 332 333 334 335 336 337 338 339 340 341 342 343 344 345 346 347 348 349 350 351 352 353 354 355 356 357 358 359 360 361 362 363 364 365 366 367 368 369 370 371 372 373 374 375 376 377 378 379 380 381 382 383 384 385 386 387 388 389 390 391 392 393 394 395 396 397 398 399 400 401 402 403 404 405 406 407 408 409 410 411 412 413 414 415 416 417 418 419 420 421 422 423 424 425 426 427 428 429 430 431 432 433 434 435 436 437 438 439 440 441 442 443 444 445 446 447 448 449 450 451 452 453 454 455 456 457 458 459 460 461 462 463 464 465 466 46

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	A	R	A	N	0	1	2	0	0	-	0	0	0	0	0	-	0	0	1	4	1	1	1	1	4	5	7	CAT	58
7	8	LICENSEE CODE						14	LICENSE NUMBER										25	LICENSE TYPE						30				

<u>0</u>	<u>1</u>		REPORT	<u>L</u>	<u>6</u>	<u>0</u>	<u>5</u>	<u>0</u>	<u>0</u>	<u>0</u>	<u>3</u>	<u>1</u>	<u>3</u>	<u>7</u>	<u>1</u>	<u>1</u>	<u>1</u>	<u>8</u>	<u>8</u>	<u>3</u>	<u>8</u>	<u>1</u>	<u>2</u>	<u>0</u>	<u>2</u>	<u>8</u>	<u>3</u>	<u>9</u>
7		8	SOURCE	60		61		DOCKET NUMBER				68	69		EVENT DATE				74	75		REPORT DATE				80		

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

On 11/18/83, during surveillance reviews being conducted as a result of corrective actions taken in regard to
LER 368-83-044/01T-0, it was determined that diesel fuel oil analysis for 6/7/83 did not meet the requirements
of Technical Specification (T.S.) 4.8.1.1.2.b for operability of Unit 2 Diesel Generator 2DG1. As a result of
failing to recognize exceeding the requirements of T.S. 4.8.1.1.2.b, the action requirements of T.S. 3.8.1.1.a
were not taken within the required time frame. Further investigation of both ANO-1 and ANO-2 fuel storage tank
data sheets revealed the following discrepancies for ANO-1: July 3, 1978, the water and sediment analysis
indicated a value of .06% (limit of .05%). A resample analysis conducted on July 5, 1978, indicated a water

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE		COMP SUBCODE		VALVE SUBCODE		80	
0 9		E 11		A 12		Z 13 Z 14		Z 15		Z 16			
7 8		9 10		11		12 13 14 15 16 17 18		19		20			
17	LER/RO REPORT NUMBER	EVENT YEAR 8 3		---		SEQUENTIAL REPORT NO. 0 2 6		OCCURRENCE CODE /		REPORT TYPE T		REVISION NO 0	
		21 22		23		24 25 26		27 28 29		30		31 32	

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
1 X 18	1 Z 19	1 Z 20	1 Z 21	1 0 1 0 1 0 1 0 22	1 Y 23	1 N 24	1 Z 25	1 Z 1 9 1 9 1 9 26
33	34	35	36	37 40	41	42	43	44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 | 0 | | Chemistry personnel did not relate the out-of-spec condition of the fuel oil analysis with a Limiting Condition

1 | 1 | | of Operation (LCO) on the diesel generators, although they did recognize that there was an out-of-spec condi-

1 | 2 | | tion. As a result, the actions required by the LCO were not taken within the allowable time frame, although a

1 | 3 | | resample was conducted as work load permitted. The procedure used for the analysis did not give explicit in-

1 | 4 | | structions or actions required when out-of-spec results were found. The diesel fuel storage tank sampling

FACILITY STATUS		% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
1	5	1	0	0	29
7	8	9	12	13	12
ANO-2 Status-H		30		C	31
44		45		46	
Review of surveillance tests				32	

ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
1	6	2	34	NA	35	NA	36
7	8	9	10	11	44	45	80

PERSONNEL EXPOSURES									
NUMBER		TYPE		DESCRIPTION					
1	7	0	0	0	37	2	38	NA	
7	8	9	11	12	13				

PERSONNEL INJURIES										80
NUMBER					DESCRIPTION					
1	8	0	0	40	NA					41
7	8	9	11	12						80

LOSS OF OR DAMAGE TO FACILITY		80
TYPE	DESCRIPTION	
1 9	2 42 NA	43

[illegible]

NAME OF PREPARER: Patrick Rogers

PHONE: (501) 964-3100

8312200062 831202
PDR ADCK 05000313
S PDR

LICENSEE EVENT REPORT

EXHIBIT A

LER No. 50-313/83-026/01T-0

Occurrence Date: 11/18/83

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (Continued)

and sediment of <.05%. June 10, 1983, the water and sediment analysis indicated a value of .06% (limit of .05%). A resample analysis conducted on June 10, 1983, indicated a water and sediment of <.05%.

For ANO-2: December 7, 1983, the water and sediment analysis indicated a value of .10% (limit of .05%). A resample analysis conducted on December 8, 1983, indicated a water and sediment of <.05%. February 2, 1983, the viscosity analysis indicated a value of 1.45 centistokes (cSt) (limit is a range of 2.0-4.3 cSt). A resample analysis conducted on February 10, 1983, indicated a viscosity of 2.55 cSt. June 7, 1983, the water and sediment analysis indicated a value of .10% (limit of .05%). A resample analysis conducted on June 13, 1983, indicated a water and sediment analysis of .05%.

There were no safety implications in that follow-up sample analyses verified that the fuel was in fact within specifications and the out-of-spec indications were a result of sampling errors. However, since the requirements of ANO-1 T.S. 4.6.1.4.e were exceeded and the action requirements of T.S. 3.7.3.c were not met, the ANO-1 discrepancies are reportable per T.S. 6.12.3.1.b. Since the requirements of ANO-2 T.S. 4.8.1.1.2.b were exceeded and the action requirements of T.S. 3.8.1.1.a were not met, the ANO-2 discrepancies are reportable per T.S. 6.9.1.8.b.

CAUSE DESCRIPTION AND CORRECTION ACTIONS (Continued)

procedures have been revised to identify Technical Specification requirements and give explicit instructions when limits are reached. Also, revisions have been made to the diesel fuel sampling procedures to address the sampling errors. A review of Chemistry and Radiochemistry Departments' procedures and Technical Specification requirements has been conducted. No similar problem with procedures other than the diesel fuel sampling procedures have been identified. A review of the procedure containing the "Master Test Control List" was also conducted and changes have been submitted to upgrade the list. Additional instructions are being given to the Chemists to insure awareness of their duties associated with Technical Specifications.



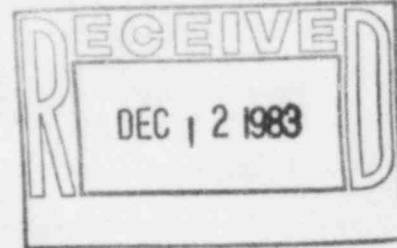
ARKANSAS POWER & LIGHT COMPANY

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December 2, 1983

1CAN128302

Mr. J. E. Gagliardo, Director
Division of Resident Reactor Projects
and Engineering Programs
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011



Subject: Arkansas Nuclear One - Unit 1
Docket No. 50-313
License No. DPR-51
Licensee Event Report
No. 83-026/01T-0

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 1 Technical Specification 6.12.3.1.b and Unit 2 Technical Specification 6.9.1.8.b, attached is the subject report concerning the failure of diesel fuel analysis to meet Technical Specification requirements.

Very truly yours,

John R. Marshall
John R. Marshall
Manager, Licensing

JRM:RJS:s1

Attachment

cc: Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

IE-29
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