

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	A	L	B	R	E	1	2	0	0	-	0	0	0	d	d	-	0	0	3	4	1	1	1	1	4			5
7	8	LIC 名字 8 字节 COD 区						14	LIC 名字 8 字节 NUM 编号						20	LIC 名字 8 字节 TYPE						26	CAT 5 字节						

CON'T

01 REPORT SOURCE L 5 0 5 0 0 0 2 5 9 7 1 1 1 2 8 3 8 1 2 0 8 8 3 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | During refuel outage, while performing SI 4.10.A.1, the interlock that prevents

03 bridge travel over the core with 1 rod withdrawn and fuel grapple extended failed

0 4 to operate. This resulted in a loss of rod block function with the platform over

05 the core (T.S. 3.10.A.1). There are no redundant systems. There was no effect on

06 the health and safety of the public.

0 9 7 8 F D 9 11 X 12 Z 13 I N S T R U 14 \$ 15 Z 16

17 LER/RO  
REPORT  
NUMBER

(17) LER/RQ REPORT NUMBER [ 8 | 3 ] [ — ] [ 0 | 6 | 6 ] [ / ] [ 0 | 3 ] [ — ] [ — ]  
 21 22 23 24 25 26 27 28 29 30 31 32

ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-6 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER		
E	18	X	19	Z	20	Z	21	0	0	Y	23	Y	24	N	25	P	B	P
33		34		35		36		37	40	41		42		43		44	45	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The lever arm on limit switch #1 (GE Part # CR 115GW307) was found to be

11 Improperly adjusted. The cause is undetermined. The lever arm was adjusted and

1 2 SI 4.10.A.1 was successfully completed. Recurrence control is under study and

13 details will be provided in a follow-up report by April 1, 1984.

FACILITY  
STATUS

POWER

OTHER STATUS

#### METHOD OF DISCOVERY

#### DISCOVERY DESCRIPTION

1	5	H	(28)	0	0	0	(29)	NA	B	(31)	Surveillance testing
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ACTIVITY	CONTENT
RELEASED	OF RELEASE

AMOUNT OF ACTIVITY

LOCATION OF RELEASE (3b)

1	6	Z	(33)	Z	(34)	NA	NA
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PERSONNEL EXPOSURES	
NUMBER	TYPE

NUMBER	TYPE	DESCRIPTION
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1	7	0	0	0	(37)	Z	(38)	NA
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PERSONNEL INJURIES

NUMBER	DESCRIPTION (41)
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1	8	0	0	0	40	NA
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LOSS OF OR DAMAGE TO FACILITY	
TYPE	DESCRIPTION

TYPE		DESCRIPTION		(43)	NA
1	9	Z	(42)		

ISSUED	DESCRIPTION	PUBLICITY
		(45)

ISSUES		DESCRIPTION		NA	
2	0	N	(44)		

NAME OF PREPARER E. T. Holder

PHONE 205 - 729-0885

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 83066 Technical Specification Involved 3.10.A.1

Reported Under Technical Specification 6.7.2.b.(2) \* Date Due NRC 12/11/83

Event Narrative:

Unit 2 was operating at 73% power, and both units 1 and 3 were in refueling outages. While performing SI 4.10.A.1 on unit 1, the interlock limit switch #1 failed to stop the bridge from traversing over the core with 1 rod withdrawn, the fuel grapple extended, and the mode switch in refuel. This resulted in a loss of the rod block function with the platform over the core (T.S. 3.10.A.1). The SI was stopped and bridge moved from over the core. The lever arm on limit switch #1 (GE Part # CR 115GW307) was found to be improperly adjusted. The cause is undetermined as the interlock switches were checked for proper condition, tightness, and operationally tested in previous steps of SI 4.10.A.1. The lever arm was adjusted and SI 4.10.A.1 was successfully completed. Recurrence control is under study and details will be provided in a follow-up report by April 1, 1984.

\* Previous Similar Events:

LER BFRO-50-259/7727  
LER BFRO-50-260-8054  
LER BFRO-50-296-8053  
LER BFRO-50-260-8301

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

\*Revision: JRP

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower 11

December 8, 1983

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW.  
Atlanta, Georgia 30303

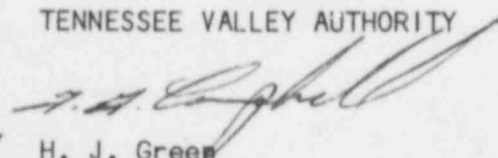
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET  
NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE  
REPORT BFRO-50-259/83066

The enclosed report provides details concerning failure of the interlock  
which provides rod block function while the refuel platform is over the  
core. This report is submitted in accordance with Browns Ferry unit 1  
Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
H. J. Green  
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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Institute of Nuclear Power Operations  
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Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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