

NRC FORM 365 (12-81) 10 CFR 50		U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT		APPROVED BY OMB 3150-0011	
CONTROL BLOCK:		(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)			
01	A L B R F 2	000-000000-000	03411111	04	05
LICENSEE CODE		LICENSE NUMBER		LICENSE TYPE CAT	
CONT					
01	L	00500026	07111283	08120883	09
REPORT SOURCE		DOCKET NUMBER		EVENT DATE REPORT DATE	
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES					
02	During startup following reactor scram, R factor (percent power/core maximum				
03	fraction limiting power density) was .967 which was less than R setpoint of				
04	1.0. T.S. 2.1.A.1.a and c require R to be greater than R setpoint.				
05	R factor was out-of-limits for 57 minutes. There was no effect on public				
06	health or safety in that the APRM scram value (calculated) was not exceeded.				
07	The .20-percent scram clamp was operable.				
08					
09	I A	X	Z	Z Z Z Z Z Z	Z Z
SYSTEM CODE		CAUSE CODE		COMP. VALVE	
SUBCODE		SUBCODE		SUBCODE	
17	83	073	03	L	0
LER/NO REPORT NUMBER		SEQUENTIAL REPORT NO.		REVISION NO.	
Y	X	Z	00000	N	Z
ACTION TAKEN		SHUTDOWN METHOD		PRIME COMP. SUPPLIER	
FUTURE ACTION		HOURS		ATTACHMENT SUBMITTED	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS					
10	This event was caused by a xenon transient due to a reactor scram and the				
11	pulling of rods to the prescram pattern. Rods were inserted to bring R back				
12	into limits. Technical specification revisions have been submitted for unit				
13	2 (similar to present unit 1 technical specifications) to allow six hours for				
14	correcting R.				
15	C	082	NA	A	Engineer Observed
FACILITY STATUS		OTHER STATUS		DISCOVERY DESCRIPTION	
16	Z	Z	NA	NA	LOCATION OF RELEASE
ACTIVITY CONTENT RELEASED		AMOUNT OF ACTIVITY			
17	000	Z	NA		
PERSONNEL EXPOSURES					
18	000	0	NA		
PERSONNEL INJURIES					
19	Z	NA			
LOSS OF OR DAMAGE TO FACILITY					
20	N	NA			
PUBLICATION ISSUED DESCRIPTION					
21	N	NA			
NAME OF PREPARER G. D. Henry					
PHONE 205/729-0845					

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 83073 Technical Specification Involved 2.1.A.1.a and c

Reported Under Technical Specification 6.7.2.b(2) * Date Due NRC 12/12/83

Event Narrative:

Units 1 and 3 were in a refueling outage. These units were unaffected by this event. Unit 2 was at 82-percent power following a reactor scram. Rod adjustments were being made to return the unit to full power. The nuclear engineer noted that calculated R (FRP/CMFLPD) was .967 which was less than the R setpoint of 1.0 prescribed in Technical Specification 2.1.A.1.a and c. There was no effect on the public health or safety as the calculated APRM scram value was not exceeded during the event. The 120-percent scram clamp was operable during the event.

The cause of the event was a xenon transient due to a reactor scram and the pulling of rods to the prescram pattern. Rods were inserted to bring R back within technical specification limits. R factor was out of limits for a total of 57 minutes which is consistent with the generic LCO (6-hour limit). Based on previous events and BWR Standard Technical Specifications, TVA has initiated changes to the technical specifications involved. These changes have been approved and issued for unit 1. When similar technical specification revisions are approved for units 2 and 3, TVA will be allowed six hours for R factor corrections on all 3 units.

* Previous Similar Events:

BFRO-50-259/80053, 80056, 80078, 82019
260/81005, 81006, 81007, 81018, 81037, 82022, 83013, 83019, 83020,
83021, 83035, 83039, 83053
296/790^3, 81018, 81040, 81041, 81045, 82038, 82048, 82062

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower 11

December 8, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW.
Atlanta, Georgia 30303

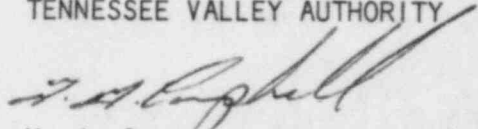
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET
NO. 5G-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE
REPORT BFRO-50-260/83073

The enclosed report provides details concerning an R factor value which was
not within technical specification limits. This report is submitted in
accordance with Browns Ferry unit 2 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY


H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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