

## LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N Y I P S 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 14 15 25 26 30 57 CAT 58

CON'T

0 1 L 6 0 5 0 0 0 2 4 7 7 1 0 2 6 8 3 8 1 1 2 5 8 3 9  
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During plant start-up following a maintenance outage the volume of water in  
0 3 the Condensate Storage Tank (CST) was reduced below the quantity required by  
0 4 Technical Specifications (3.4.A.3) on two different occasions. The volume was  
0 5 restored above minimum within the time limit permitted (3.4.B). Health and safety  
0 6 of the public were not affected. Previous Events AO 75-10, RO 76-16, 76-17,  
0 7 LER 78-29, 79-10, 79-10, 79-22 & LER 82-50  
0 8  
0 9

0 9 H H 11 X 12 Z 13 A C C U M U 14 Z 15 Z 16  
7 8 9 10 11 12 13 18 19 20  
17 LFR/RO REPORT NUMBER 8 3 21 22 23 24 25 26 27 28 29 30 31 32  
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER  
X 18 Z 19 Z 20 Z 21 0 0 0 0 Y 23 N 24 N 25 C 3 1 0 0  
33 34 35 36 37 40 41 42 43 44 47

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The reduced volume in CST below the Technical Specification was the result of  
1 1 efforts required to maintain concentrations of sodium chloride within limits in the  
1 2 steam generators. This effort involved discharging hotwell condensate and supplying  
1 3 make-up water from CST. After these efforts the CST level was returned to  
1 4 normal.  
7 8 9 80

1 5 C 28 0 0 0 29 NA 30 A 31 Control Room Observation 32  
7 8 9 10 12 13 44 45 46 80

1 6 Z 33 Z 34 NA 35 NA 36  
7 8 9 10 11 44 45 80

1 7 0 0 0 37 Z 38 NA 39  
7 8 9 10 11 12 13 80

1 8 0 0 0 40 NA 41  
7 8 9 10 11 12 80

1 9 Z 42 NA 43  
7 8 9 10 80

2 0 N 44 NA 45  
7 8 9 10 80

NAME OF PREPARER Gary Hinrichs

PHONE: 914-526-5543

ATTACHMENT

Docket No. 50-247  
LER 83-040/03L-0

Consolidated Edison Co. of New York, Inc.  
Indian Point Station Unit 2

On October 26, 1983, while starting up the plant following a maintenance outage, the volume of water in the Condensate Storage Tank (CST) was reduced below that required by the Technical Specification. The value specified in Section 3.4.A.3 is 360,000 gallons which is equivalent to 19 feet tank level. The chloride concentration of the water in the condenser hotwells increased to unacceptable levels due to condenser tube leaks. The CST level dropped below 19 feet due to increased blowdown and condensate system flushing while providing make-up to the steam generators from the CST.

Again on October 27, 1983 during the plant start-up, the level in the CST dropped to 17 feet during flushing of the condensate system.

In both instances, the CST water level was returned above 19 feet within the time limit permitted to Technical specification 3.4.B. A portable ion bed demineralizer was installed to increase the available make-up water supply in addition to that normally supplied from the water factory. Water was also transferred from Unit 3 to supplement make-up.

Previous similar events have been reported in A.O. 75-10, R.O. 76-16 and 76-17, and LERs 78-29, 79-10, 79-22 and 82-50.

John D. O'Toole  
Vice President

Consolidated Edison Company of New York, Inc.  
4 Irving Place, New York, NY 10003  
Telephone (212) 460-2533

November 25, 1983

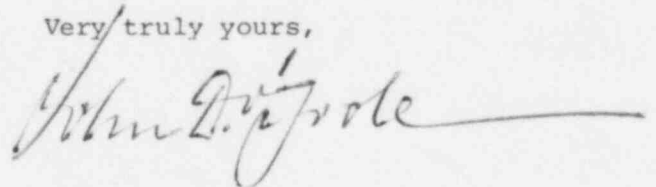
Re: Indian Point Unit No. 2  
Docket No. 50-247  
LER-83-040/03L-0

Dr. Thomas E. Murley,  
Regional Administrator-Region I  
U. S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pa. 19406

Dear Dr. Murley:

The attached Licensee Event Report LER-83-040/03L-0 is hereby submitted in accordance with the requirements of Technical Specification 6.9.1.7. This event is of the type described in Technical Specification 6.9.1.7.2.b.

Very truly yours,



attach.

cc: Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Mr. Thomas Foley, Senior Resident Inspector  
U. S. Nuclear Regulatory Commission  
P. O. Box 38  
Buchanan, New York 10511

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