

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

December 8, 1983

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
Reactor Trip Breaker Automatic
Shunt Trip

REFERENCE: Letter dated November 23, 1983,
from T. M. Novak to O. W. Dixon, Jr.

Dear Mr. Denton:

The referenced letter identified three confirmatory items which are included in the safety evaluation of the reactor trip breaker automatic shunt trip design modification. The three confirmatory items and the associated response are as follows:

1. Confirm that the environmental conditions defined by the Westinghouse Owner's Group (WOG) generic design are applicable to your application.

A review of the environmental parameters defined by the WOG generic design was performed. It has been evaluated that the conditions defined by the WOG generic design are applicable to the installation of the automatic shunt trip design modification at the Virgil C. Summer Nuclear Station.

2. Confirm that your surveillance test procedure required opening the "block auto shunt trip" switch contacts during the test of the control room manual trip switch to confirm the operability of the manual shunt trip circuit independent of the automatic shunt trip circuit.

The appropriate surveillance test procedures require that the "block automatic shunt trip" switch contacts are opened during the test of the control room manual trip switch.

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Mr. Harold R. Denton
Reactor Trip Breaker Automatic
Shunt Trip
Page Two
December 8, 1983

3. Provide proposed Technical Specifications for the modification.

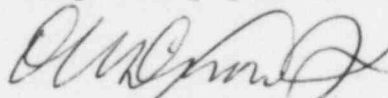
Provided as Enclosure 1 are proposed changes to the Technical Specifications concerning the surveillance testing of the reactor trip breakers. Upon Nuclear Regulatory Commission (NRC) approval, these revised Technical Specifications will supersede the surveillance testing requirements outlined in the letters from O. W. Dixon, Jr. to H. R. Denton, dated May 12, 1983, and June 8, 1983.

South Carolina Electric & Gas Company (SCE&G) has determined that a finding of no significant hazards is justified in that additional requirements are being added to the Technical Specifications. This change has been reviewed and approved by the Plant Safety Review Committee (PSRC) and the Nuclear Safety Review Committee (NSRC). A check in the amount of Four Thousand Dollars (\$4,000.00) is enclosed for processing this change.

SCE&G is a participating member of the Westinghouse Owner's Group, which is in the process of developing generic requirements for reactor trip breaker testing and maintenance. It is expected that the enclosed proposed Technical Specifications will not be approved for implementation until these final generic guidelines are issued, at which time SCE&G may elect to amend this proposal. Until final Technical Specifications are approved and issued, reactor trip breaker maintenance and testing will continue in accordance with commitments described in letters from O. W. Dixon, Jr. to H. R. Denton, dated May 12, 1983, and June 8, 1983.

If you have any questions, please contact us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

ARK:AMP:OWD/fjc/dwf

cc: See Page Three

Mr. Harold R. Denton
Reactor Trip Breaker Automatic
Shunt Trip
Page Three
December 8, 1983

cc: V. C. Summer
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