NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE (9–14)	(2)	LICENSE NUMBER (15–25)	(3)	LICENSE TYPE (26–30)	(4)	CAT (57–58)	(5)
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CON'T 0 1	REPORT SOURCE (60)	(6)	DOCKET NUMBER (61–68)	(7)	EVENT DATE (69–74)	(8)	REPORT DATE (75–80)	(9)
0 1	L		0 5 0 0 0 2 7 2		1 1 1 0 8 3		1 2 0 2 8 3	

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 On November 10, 1983, RCS unidentified leakage was 1.1 GPM. Maximum allowed by the

0 3 technical specifications is 1.0 GPM and Action Statement 3.4.6.2b was entered. Un-

0 4 identified leakage decreased below 1.0 GPM with no action required and no undue risk

0 5 to the health or safety of the public was therefore involved in this occurrence. The

0 6 event constituted operation in a degraded mode permitted by a limiting condition for

0 7 operation and is reportable in accordance with Technical Specification 6.9.1.9b.

0 8

0 9	SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP. SUBCODE (15)	VALVE SUBCODE (16)
	C I	X	Z	Z Z Z Z Z Z	Z	Z

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 5 5	/	0 3	L	—	0

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
Z	Z	Z	Z	0 0 0 0	Y	N	Z	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The amount of humidity, condensation and containment sump in-leakage varies with

1 1 changes of Containment Fan Coil Unit (CFCU) configuration and speed. A recent

1 2 change in CFCU configuration apparently caused unidentified leakage, which was

1 3 previously close to the limit, to temporarily exceed 1.0 GPM.

1 4

1 5	FACILITY STATUS (28)	% POWER (29)	OTHER STATUS (30)	METHOD OF DISCOVERY (31)	DISCOVERY DESCRIPTION (32)
	E	1 0 0	N/A	B	Operator Surveillance

1 6	ACTIVITY RELEASED (33)	CONTENT OF RELEASE (34)	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
	Z	Z	N/A	N/A

PERSONNEL EXPOSURES

1 7	NUMBER (37)	TYPE (38)	DESCRIPTION (39)
	0 0 0	Z	N/A

PERSONNEL INJURIES

1 8	NUMBER (40)	DESCRIPTION (41)
	0 0 0	N/A

8312130156 831202
PDR ADOCK 05000272
S PDR

LOSS OF OR DAMAGE TO FACILITY (43)

1 9	TYPE (42)	DESCRIPTION
	Z	N/A

PUBLICITY

2 0	ISSUED (44)	DESCRIPTION (45)	NRC USE ONLY
	N	N/A	

NAME OF PREPARER J. L. Rupp PHONE: (609) 339-4309

GPO 917-926

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

December 2, 1983

Dr. Thomas E. Murley
Regional Administrator
USNRC
Region 1
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Dr. Murley:

LICENSE NO. DPR-70
DOCKET NO. 50-272
REPORTABLE OCCURRENCE 83-055/03L

Pursuant to the requirements of Salem Generating Station Unit No. 1, Technical Specifications, Section 6.9.1.9b, we are submitting Licensee Event Report for Reportable Occurrence 83-055/03L. This report is required within thirty (30) days of the occurrence.

Sincerely yours,

J. M. Zupko, Jr.
General Manager -
Salem Operations

JR:kll

CC: Distribution

IE22
1/1

95-2189 (20M) 11-81

Report Number: 83-055/03L

Report Date: 12-02-83

Occurrence Date: 11-10-83

Facility: Salem Generating Station Unit 1
Public Service Electric & Gas Company
Hancocks Bridge, New Jersey 08038

IDENTIFICATION OF OCCURRENCE:

Reactor Coolant System - Unidentified Leakage - Out-Of-Specification

This report was initiated by Incident Report 83-202

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1 - Rx Power 100 % - Unit Load 1147 MWe

DESCRIPTION OF OCCURRENCE:

At 0556 hours, November 10, 1983, during the performance of Surveillance Procedure SP(O)4.4.6.2, it was determined that the amount of unidentified containment sump in-leakage was 1.1 GPM. The Technical Specifications allow a maximum of 1.0 GPM unidentified leakage from the Reactor Coolant System (RCS). An operator was immediately sent into containment to determine the source of leakage and Technical Specification Action Statement 3.4.6.2b was entered. Some condensation was apparent in the containment; in addition, a small amount of leakage was observed coming from inside of the bioshield. Containment sump in-leakage was monitored, documented and unidentified leakage decreased to less than 1.0 GPM with no corrective action required. At 1003 hours, November 10, 1983, Action Statement 3.4.6.2b was terminated.

APPARENT CAUSE OF OCCURRENCE:

The operating containment fan coil units are alternated on a periodic basis. The configuration of the operating units cause different air flow patterns in the containment, resulting in condensate which is caused by any leakage present to temporarily form until equilibrium conditions within the containment are reached. The amount of humidity, condensation and containment sump in-leakage vary with changes of CFCU configuration and speed. A recent change in CFCU configuration apparently caused unidentified leakage, which was previously close to the limit, to temporarily exceed 1.0 GPM.

ANALYSIS OF OCCURRENCE:

Technical Specification 3.4.6.2 requires that RCS leakage be limited to:

a. No pressure boundary leakage,

b. Ten (10) GPM identified leakage, and

ANALYSIS OF OCCURRENCE: (cont'd)

c. One (1) GPM unidentified leakage.

The RCS leakage limits are based on ensuring the ability to detect leakage from the reactor coolant boundary. The one (1) GPM value is sufficiently low to ensure early detection of additional leakage; the ten (10) GPM identified leakage limitation provides allowance for a limited amount of leakage from known sources whose presence will not interfere with the detection of unidentified leakage by the leak detection systems; and, pressure boundary leakage of any magnitude is unacceptable since it may be indicative of an impending gross failure of the pressure boundary.

Action Statement 3.4.6.2b requires:

> With any RCS leakage greater than 10 GPM identified or greater than 1 GPM unidentified, reduce the leakage to within limits within 4 hours or be in cold shutdown within the following 30 hours. Any pressure boundary leakage requires being in hot standby within 6 hours and in cold shutdown within the following 30 hours.

Unidentified leakage decreased to less than 1.0 GPM within the time specified by the action requirement and with no corrective action required. All Reactor Coolant Leakage Detection Systems were operable throughout the occurrence and identified leakage never exceeded 10 GPM. No undue risk to the health or safety of the public was involved in the occurrence. The event constituted operation in a degraded mode permitted by a limiting condition for operation and is therefore reportable in accordance with Technical Specification 6.9.1.9b.

CORRECTIVE ACTION:

As previously stated, unidentified leakage decreased to less than 1.0 GPM within the time specified by the action requirement. Technical Specification Action Statement 3.4.6.2b was terminated at 1003 hours, November 10, 1983.

Although unidentified leakage was less than the 1.0 GPM allowed by the technical specifications, subsequently, at 0708 hours, November 11, 1983, the unit was shutdown to identify and repair the leaks within the containment.

FAILURE DATA:

Not Applicable

Prepared By J. Rupp

General Manager -
Salem Operations

SORC Meeting No. 83-147