

CONTROL BLOCK:

| | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
|---|---|---|---------------|---|---|---|---|---|----|----|----------------|---|---|---|---|---|---|---|---|---|----|----|--------------|---|---|--|--|----|----|-----|----|
| 0 | 1 | P | A | B | V | S | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | | | 5 | | | |
| 7 | 8 | 9 | LICENSEE CODE | | | | | | 14 | 15 | LICENSE NUMBER | | | | | | | | | | 25 | 26 | LICENSE TYPE | | | | | 30 | 57 | CAT | 58 |

REPORT SOURCE L 6 0 5 0 0 0 3 3 4 7 1 0 2 6 8 3 8 1 1 1 7 8 3 9

60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

0 2 On 10/26/83, during the performance of a surveillance test to verify the

0 3 operability of the LB quench spray pump, the 480 volt breaker for the

0 4 pump could not be exercised. The manual handle for the breaker had

0 5 disengaged from the closing device and jammed the breaker in the tripped

0 6 position. There were no safety implications to the public because the

0 7 redundant quench spray train was operable and available.

| | |
|---|---|
| 0 | 8 |
|---|---|

| | | | | | | | | | | | | | | | | | | |
|----------------------|----|---------------|----|-----------------------|----|-----------------|----|----------------|----|----------------------|----|------------------|----|----------------------|----|---|---|---|
| 09 | | SYSTEM CODE | | CAUSE CODE | | CAUSE SUBCODE | | COMPONENT CODE | | | | COMP. SUBCODE | | VALVE SUBCODE | | | | |
| 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 | 17 | 18 | 19 | 20 | 21 | 22 | | | |
| | | S | R | X | | Z | | C | K | T | B | R | K | A | Z | | | |
| 17 | | EVENT YEAR | | SEQUENTIAL REPORT NO. | | OCCURRENCE CODE | | REPORT TYPE | | REVISION NO. | | | | | | | | |
| 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 33 | 34 | 35 | 36 | 37 | 38 | | | |
| LER/RO REPORT NUMBER | | 8 | 3 | | 0 | 3 | 4 | | 0 | 3 | | L | | 0 | | | | |
| ACTION TAKEN | | FUTURE ACTION | | EFFECT ON PLANT | | SHUTDOWN METHOD | | HOURS | | ATTACHMENT SUBMITTED | | NPRD-4 FORM SUB. | | PRIME COMP. SUPPLIER | | | | |
| 39 | 40 | 41 | 42 | 43 | 44 | 45 | 46 | 47 | 48 | 49 | 50 | 51 | 52 | 53 | 54 | | | |
| B | X | | Z | | Z | | | 0 | 0 | 0 | 0 | Y | Y | L | G | O | R | O |

1 0 The cause for this incident is unknown, it is judged that excessive

1 1 force may have been used on the breaker. No previous work was performed

1 2 on this breaker after the last successful completion of its surveillance

1 3 test until this failure. A letter will be issued to all Operations personnel

informing them of this incident and its speculated cause.

| | | | | | | | | | | | | | | |
|---------------------|----|--------|---------------------|----------|----------|-------------------------------|----------|------|--|------------------------|--|-----------------------|--|--|
| FACILITY STATUS | | | % POWER | | | OTHER STATUS | | | METHOD OF DISCOVERY | | | DISCOVERY DESCRIPTION | | |
| 1 | 5 | E (28) | 1 | 0 | 0 (29) | N/A | | | B (31) | Surveillance Test (32) | | | | |
| ACTIVITY CONTENT | | | RELEASED OF RELEASE | | | AMOUNT OF ACTIVITY | | | LOCATION OF RELEASE | | | | | |
| 1 | 6 | Z (33) | Z (34) | N/A (35) | | | N/A (36) | | | | | | | |
| PERSONNEL EXPOSURES | | | PERSONNEL INJURIES | | | LOSS OF OR DAMAGE TO FACILITY | | | PUBLICITY | | | | | |
| 1 | 7 | 0 | 0 | 0 (37) | Z (38) | N/A (39) | | | 8312130046 831118 PDR ADOCK 05000334 S PDR | | | | | |
| 1 | 8 | 0 | 0 | 0 (40) | N/A (41) | | | IE22 | | | | | | |
| 1 | 9 | Z (42) | N/A (43) | | | N/A (44) | | | NRC USE ONLY | | | | | |
| 1 | 10 | N (45) | N/A (46) | | | N/A (47) | | | N/A (48) | | | | | |

NRC USE ONLY

Robert J. Druga, Chief Engineer

412/643-1264

Attachment to LER 83-034/03L
Beaver Valley Power Station
Duquesne Light Company
Docket No. 50-334

The 480 Volt breaker was manufactured by General Electric
Company.

Model No.: AK-3A-25



Duquesne Light

Nuclear Division
P.O. Box 4
Shippingport, PA 15077-0004

Telephone (412) 393-6000

November 18, 1983
ND1SS1:1016

Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
LER 83-034/03L

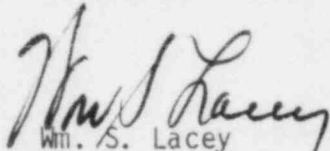
Dr. Thomas E. Murley
Regional Administrator
United States Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, PA 19406

Dear Dr. Murley:

In accordance with Appendix A, Beaver Valley Technical Specifications,
the following Licensee Event Report is submitted:

LER 83-034/03L, Technical Specification 3.6.2.1, Containment Quench
Spray System.

Very truly yours,


Wm. S. Lacey
Station Superintendent

Attachment

FER
1/1

Dr. T. E. Murley
November 18, 1983
ND1SS1:1016
Page two

cc: Director of Management & Program Analysis
United States Nuclear Regulatory Commission
Washington, D.C. 20555

C. A. Roteck, Ohio Edison

Director, Office of Inspection and Enforcement Headquarters
United States Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. Peter Tam, BVPS Licensing Project Manager
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