

CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)01 CASO S 3 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 1 4 5  
7 8 9 LICENSE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 37 CAT 38

CONT

31 REPORT SOURCE L 6 0 5 0 0 0 3 6 2 7 1 1 0 5 8 3 8 1 2 0 1 8 3 9  
7 8 9 DOCKET NUMBER 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On 11/15/83 at 1620, with Unit 3 in Mode 3, AFW pump 3P-140 was declared  
03 inoperable to replace a broken torque switch on the motor operator for  
04 steam inlet valve 3HV-4716, which was preventing the valve from being  
05 closed from the Control Room. In accordance with LCO 3.7.1.2, Action  
06 Statement 'a', action was initiated to restore 3P-140 to operable status  
07 within 72 hours. There was no impact on the health and safety of plant  
08 personnel or the public.

09 SYSTEM CODE H H 11 CAUSE CODE B 12 CAUSE SUBCODE B 13 COMPONENT CODE I N S T R U 14 S 15 VALVE SUBCODE Z 16  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

17 LER/RO REPORT NUMBER 8 3 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

18 ACTION TAKEN A 18 Z 19 FUTURE ACTION Z 20 EFFECT ON PLANT Z 21 SHUTDOWN METHOD Z 22 HOURS 0 0 0 0 23 ATTACHMENT SUBMITTED N 24 NPRO-4 FORM SUB. N 25 PRIME COMP. SUPPLIER A 26 COMPONENT MANUFACTURER L 2 0 0 27

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Examination of the torque switch determined that an electrical contact  
11 was apparently overtightened during manufacturing, resulting in early  
12 failure of the switch. The torque switch was replaced and 3P-140 was  
13 declared operable at 0430 on 11/06/83. This is considered an isolated  
14 occurrence and no further corrective action is planned.

15 FACILITY STATUS B 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Operator Observation 32  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

16 ACTIVITY CONTENT Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

17 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

18 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

19 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

20 PUBLICITY ISSUED DESCRIPTION N 44  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

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PDR ADOCK 05000362  
S PDR

NRC USE ONLY

NAME OF PREPARER J. G. HAYNES

PHONE 714-492-7700

*Southern California Edison Company*

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES  
STATION MANAGER

December 1, 1983

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(714) 492-7700

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REGION V

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U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region V  
1450 Maria Lane, Suite 210  
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-362  
30-Day Report  
Licensee Event Report No. 83-089  
San Onofre Nuclear Generating Station, Unit 3

Pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, this submittal provides the required 30-day written report and a copy of the Licensee Event Report (LER) form for an occurrence involving Limiting Condition for Operation (LCO) 3.7.1.2 associated with the Auxiliary Feedwater (AFW) System.

On November 5, 1983, at 1620, with Unit 3 in Mode 3, AFW pump 3P-140 was declared inoperable to replace a broken torque switch on the motor operator for steam inlet valve 3HV-4716, which was preventing the valve from being closed from the Control Room. In accordance with LCO 3.7.1.2, Action Statement 'a', action was initiated to restore 3P-140 to operable status within 72 hours.

Upon examination of the torque switch, it was determined an electrical contact was apparently overtightened during manufacturing, resulting in early failure of the switch. After replacement of the torque switch, 3HV-4716 was returned to service, and 3P-140 was declared operable at 0430 on November 6, 1983. The broken torque switch is considered an isolated occurrence, and no further corrective action is planned. There was no impact on the health and safety of plant personnel or the public associated with this event.

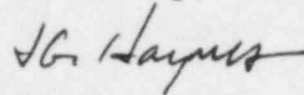
1/1

IE-22

December 1, 1983

If you require any additional information, please so advise.

Sincerely,



Enclosure: LER No. 83-089

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)  
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement

U. S. Nuclear Regulatory Commission  
Division of Technical Information and Document Control

Institute of Nuclear Power Operations (INPO)