

CONTROL BLOCK:

PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION

CONT

REPORT SOURCE L 6 0 5 0 0 0 2 6 0 1 0 2 9 8 3 4 1 1 2 5 8 3 7

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (C)

During normal operation on unit 2, the operator observed RHR Loop II flow instrumentation reading upscale. A 24 hour L.C.O. was entered. T.S. 3.5.B.4 requires 2 RHR Loops to be fully operational. Both FI-74-64 and FR-74-64, which indicate and record RHR Loop II System Total Flow, respectively, were returned to service in less than 24 hours. There was no effect on public health or safety. Loop I RHR was fully operable.

SYSTEM COOP		CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE					COMP SUBCODE	VALVE SUBCODE				
1	0	C	F	E	I	N	S	T	R	U	R	Z		
7	8	9	10	11	12	13	14	15	16	17	18	19		
LEAD REPORT NUMBER		EVENT YEAR	SEQUENTIAL REPORT NO	OCCURRENCE CODE		REPORT TYPE	REVISION NO							
17	8	3	0	6	7	0	3	L	0					
20	21	22	23	24	25	26	27	28	29					
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTER	FROM FORM SUB	PRIME COMP SUPPLIER	COMPONENT MANUFACTURER						
D	Z	Z	Z	0	0	0	Y	N	L	G	0	8	0	
30	31	32	33	34	35	36	37	38	39	40	41	42	43	44

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 1 The problem was caused by a short in the G.E. Type 531 recorder (FR-74-64)  
1 2 which drove both indicator and recorder upscale. This affected RHR Loop II  
1 3 operability. The recorder was repaired and the loop was returned to service in  
1 4 less than 24 hours. This is considered a random event and no further recurrence  
1 5 control is required.

FACILITY STATUS ☐ POWER ☐ OTHER STATUS ☐ METHOD OF DISCOVERY ☐ DISCOVERY DESCRIPTION ☐

1 5 E 20 1 0 0 29 N/A A 31 Operator Observed

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY ☐ LOCATION OF RELEASE ☐

1 6 Z 32 Z 34 N/A N/A

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION ☐

1 7 0 0 0 37 Z 38 N/A

PERSONNEL INJURIES NUMBER DESCRIPTION ☐

1 8 0 0 0 40 N/A

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION ☐

1 9 Z 42 N/A

PUBLICITY ISSUED DESCRIPTION ☐

2 0 N 44 N/A

8312080117 831125  
 PDR ADDCK 05000260  
 S PDR

N/A

FE22 11

NRC USE ONLY

NAME OF PREPARED **G. T. Chambers**

(205) 729-0626

LER SUPPLEMENTAL INFORMATION

BFRO-50- 60/ 83067 Technical Specification Involved 3.5.B.4

Reported Under Technical Specification 6.7.2.b.(2) Date Due NRC 11/28/83

Event Narrative:

Units 1 and 3 were in refueling outages. Unit 2 was operating normally at 100% power. Only unit 2 was affected by this event. During normal operation on unit 2, the operator observed RHR Loop II flow instrumentation reading upscale. A 24 hour L.C.O. was entered. T.S. 3.5.B.4 requires 2 RHR loops to be fully operational. FI-74-64 and FR-74-64 indicate and record RHR Loop II System Total Flow, respectively. The problem was caused by a short in a G.E. Type 531 recorder which caused both the recorder and the indicator to read upscale. This would have affected operability of the minimum flow valve and therefore RHR Loop II operability. The recorder was repaired and the loop was returned to service in less than 24 hours. There was no effect on public health or safety. Loop I RHR was fully operable.

This is considered a random event and no further recurrence control is required.

\* Previous Similar Events:

BFRO-50-260/83064, 83068

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

\*Revision: JRR

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower 11

83 NOV 30 A 7:22

November 25, 1983

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW.  
Atlanta, Georgia 30303

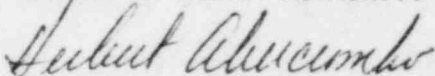
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET  
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE  
REPORT BFR0-50-260/83067

The enclosed report provides details concerning residual heat removal  
Loop 11 flow instrumentation that was reading upscale. This report is  
submitted in accordance with Browns Ferry unit 2 Technical Specification  
6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



H. J. Green  
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Records Center  
Institute of Nuclear Power Operations  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

IE 22  
1/1