

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT
/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /1/1/0/6/8/3/ (8) /1/1/2/3/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On November 6, 1983, with Unit 1 at 100 percent thermal power, LI-1494, Steam /
/0/3/ / Generator "C" Channel I Level Indication drifted high out of the specified toler- /
/0/4/ / ance of 5 percent from the other two channels. Steam Generator Level Indication /
/0/5/ / inputs signals to the Reactor Trip Circuitry and Auxiliary Feedwater Pump Start /
/0/6/ / Logic. Since the channel was placed in trip within one hour the health and /
/0/7/ / safety of the public were not affected. This event is contrary to T.S. 3.3.1.1 /
/0/8/ / and reportable pursuant to T.S. 6.9.1.9.b. /

SYSTEM CAUSE CAUSE COMP. VALVE
CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE

/0/9/ /I/A/ (11) /E/ (12) /B/ (13) /V/A/L/V/E/X/ (14) /E/ (15) /D/ (16)
LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION
REPORT NO. TYPE NO.
(17) NUMBER /8/3/ /-/ /0/7/4/ / / /0/3/ /L/ /-/ /0/

ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER

/B/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /Y/ (24) /A/ (25) /V/1/3/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / The channel failure was caused by a body to bonnet leak of the reference leg root /
/1/1/ / valve, which decreased pressure on the reference leg side thus causing a higher /
/1/2/ / level indication on the channel. The valve leak was stopped. There are no /
/1/3/ / generic implications of this event. /
/1/4/ /

FACILITY METHOD OF
STATUS %POWER OTHER STATUS (30) DISCOVERY DISCOVERY DESCRIPTION (32)
/1/5/ /E/ (28) /1/0/0/ (29) / NA / /A/ (31) / Operator Observation /

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES
NUMBER TYPE DESCRIPTION (39)
/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES
NUMBER DESCRIPTION (41) 8312080106 823112
/1/8/ /0/0/0/ (40) / NA / PDR ADOCK 05000388
S PDR

LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
/1/9/ /Z/ (42) / NA /

PUBLICITY
ISSUED DESCRIPTION (45)
/2/0/ /N/ (44) / NA /

NRC USE ONLY

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

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Vepco

Recd
VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION
P. O. BOX 402
MINERAL, VIRGINIA 23117

83 NOV 30 P 1:05

November 23, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-160
NO/CLF: 11
Docket No. 50-338
License No. NPF-4

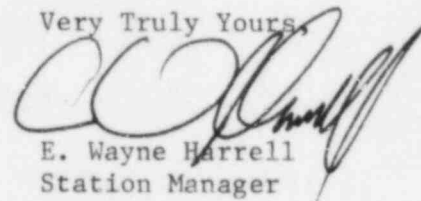
Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 1.

Report No.	Applicable Technical Specifications
LER 83-074/03L-0	T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,


E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
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U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

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